

# Characterization of the helicon plasma generated inside the Cybele negative ion source with different magnetic field configurations

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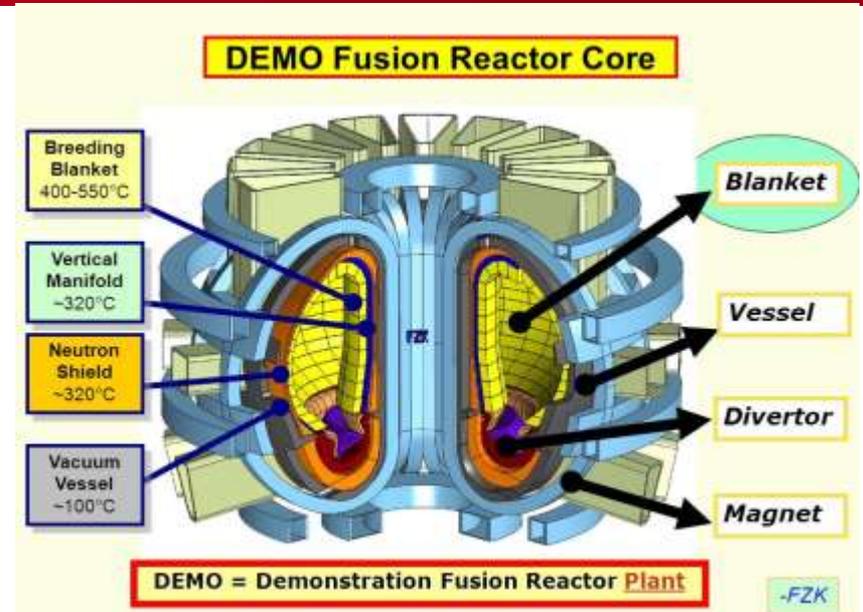
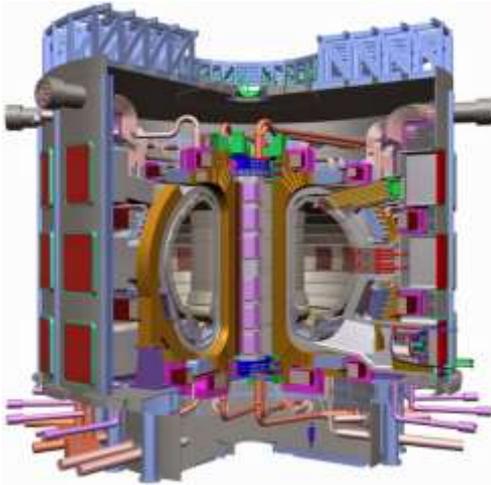
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# ITER 2030

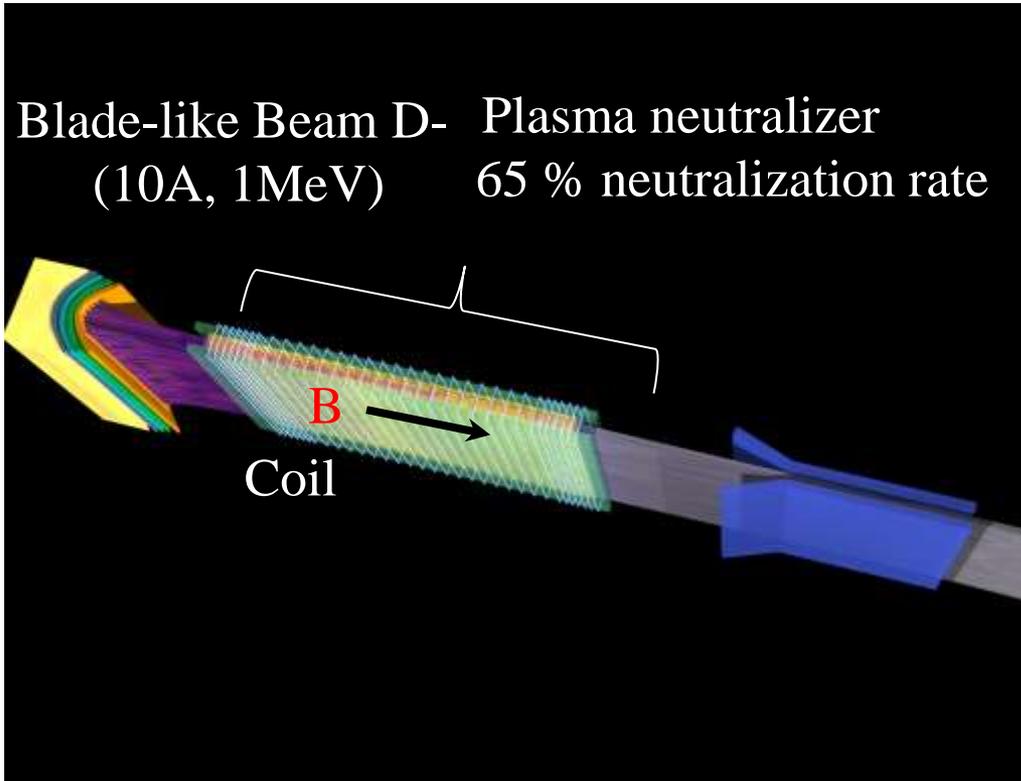


No electricity production  
 Plasma heating: NBI: 2\*17MW D° at 1MeV  
 NBI Expected efficiency :<28% [1]  
 The ITER NBI is under construction RFX testbed (MITICA) commissioning in 2023

500MW of electrical power on the net  
**DEMO1: pulsed reactor**  
 NBI: ~ 50MW D° at 1 MeV  
 Overall efficiency > 40 % [2]  
**DEMO2: Steady state**  
 NBI: ~110MW D°, 1-2 MeV (current drive)  
 Overall efficiency > 60 %

[1] R S Hemsworth *et al*, Overview of the design of the ITER heating neutral beam injectors, 2017 *New J. Phys.*  
 [2] P Sonato et al, 2016, Conceptual design of the beam source for the DEMO NBI , *New J. Phys.* 18 125002

## NBI: with plasma neutralizer



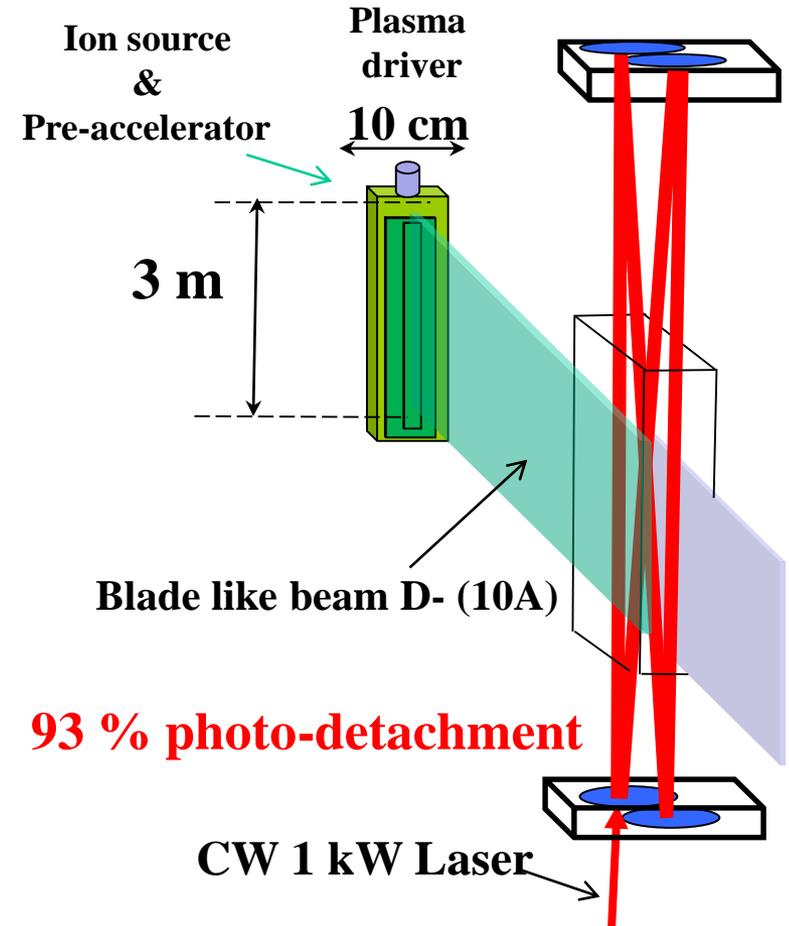
### Potential advantages of blade-like beams:

Reduce the gas load along the beam line

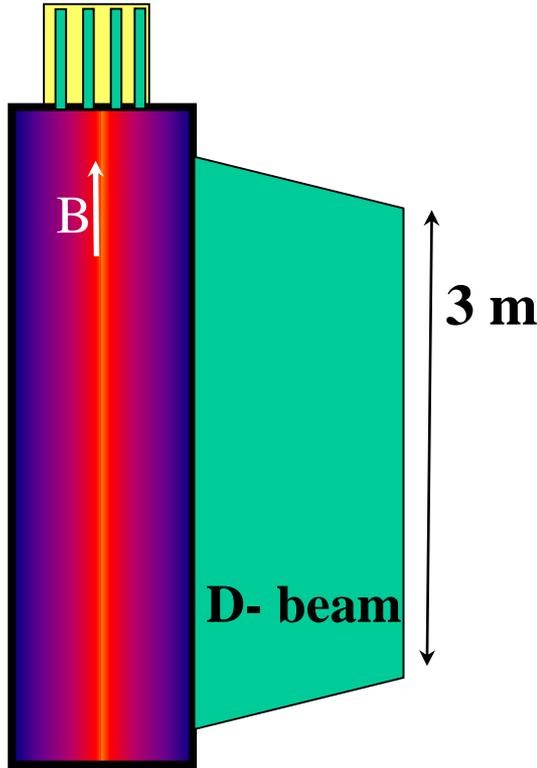
Increase the overall injector efficiency

Essential for plasma neutralizer and photoneutralizer

## with photoneutraliser [3]



RF Plasma driver

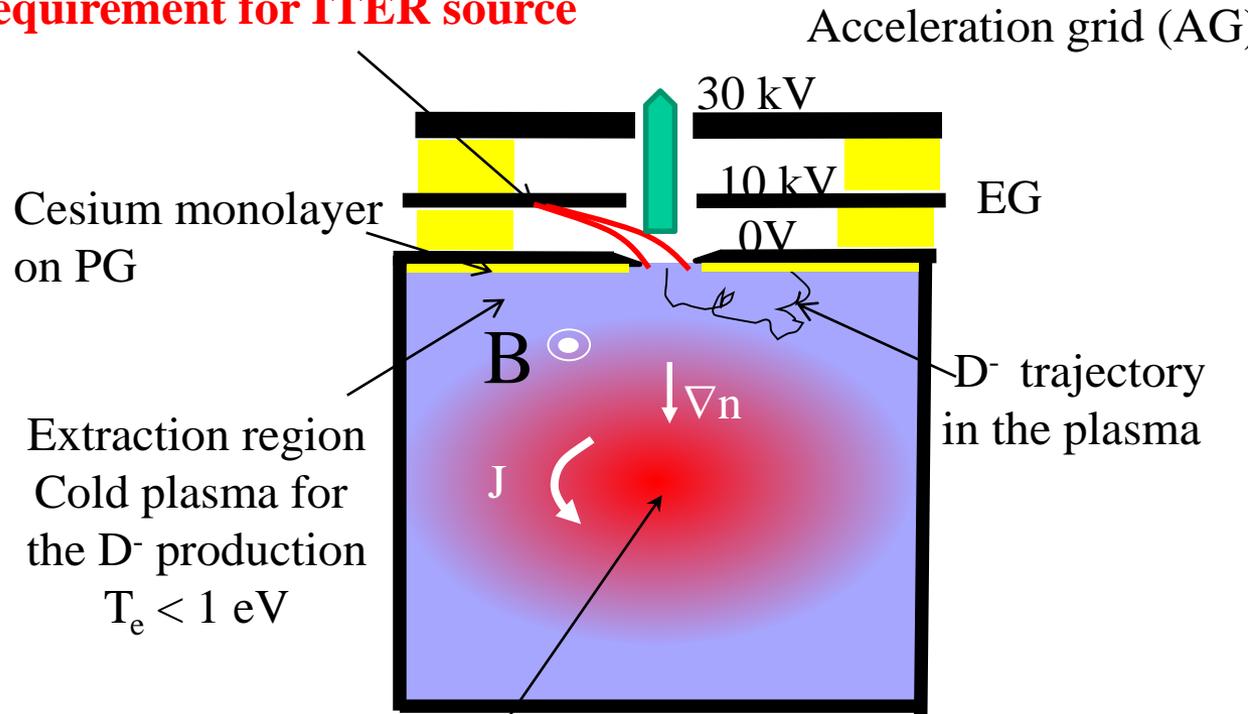


magnetized  
Plasma  
column

Side view

## Ion Source with 30 kV acceleration

Co-extracted electrons  
More than 1 e<sup>-</sup> per D<sup>-</sup>  
requirement for ITER source



Hot and dense plasma core  
 $T_e \sim 10 \text{ eV}, n_e \sim 10^{18} \text{ m}^{-3}$

Horizontal  
cross section

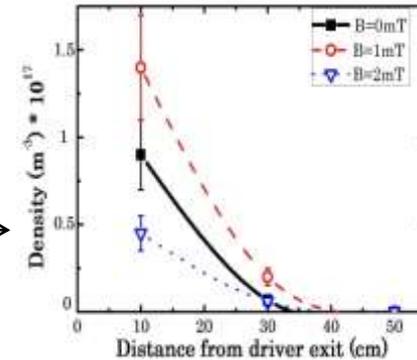
1) 2014 **Filamented cathode** [3]

- ) plasma vertically uniform along the vertical axis,
- ) peak  $N_e \sim 4 \cdot 10^{17} \text{m}^{-3}$  and  $T_e \sim 9 \text{eV}$

But, not relevant for Cs operation, due to the pollution by W

2) 2016 **ICP plasma driver**: (results presented NIBS 2016)

Plasma density drops rapidly along the vertical direction at the driver exit => **Plasma non-uniform**

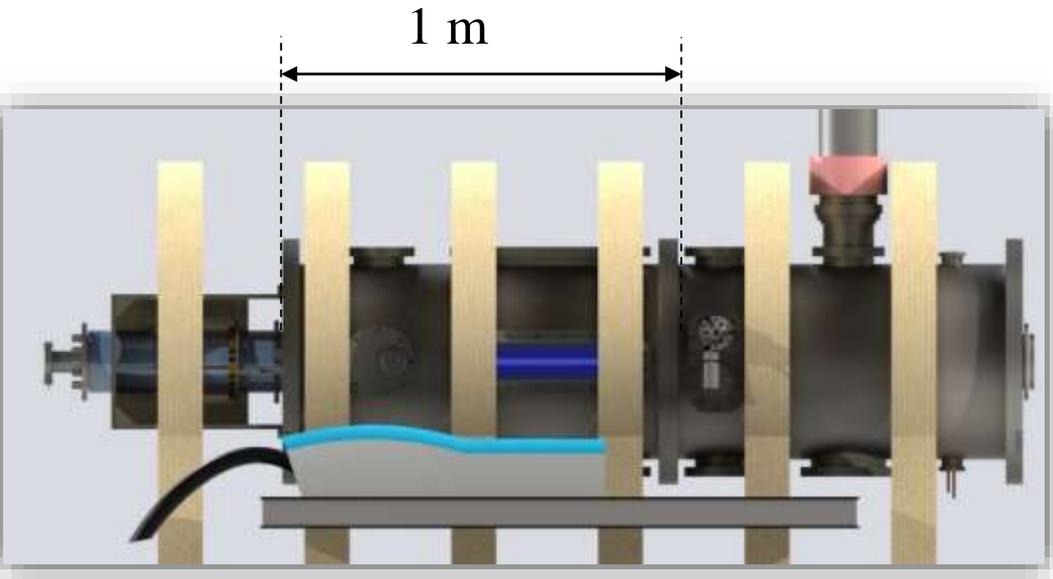
3) Since 2017 test of the **Helicon driver** developed by EPFL (see previous talk **WO8 R.Agnello**)

Operating conditions relevant for NI source:

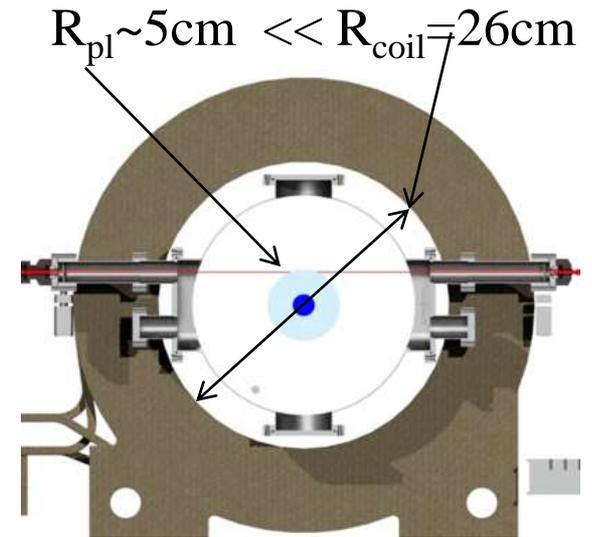
- Low B-field ( $\sim 10 \text{mT}$ )
- Quite Uniform plasma column (along 1.5m)
- High density in the center ( $> 10^{18} \text{m}^{-3}$ )
- Low  $T_e$  on the edge for NI production ( $\sim 1\text{-}2 \text{eV}$ )

**But, RAID geometry does not allow extraction of a long blade-like negative ion beam**



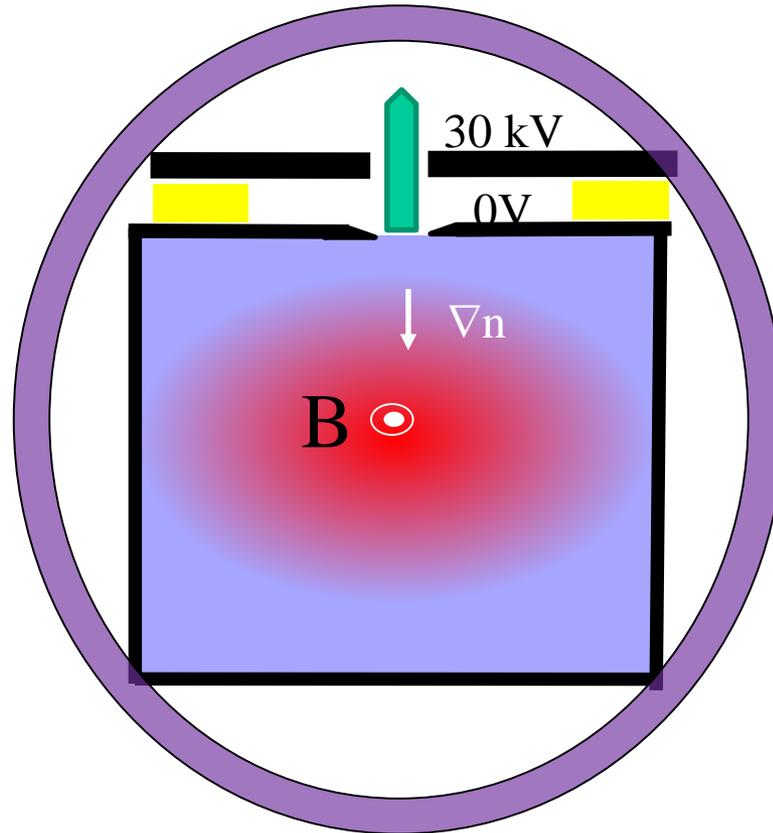


**External magnets**

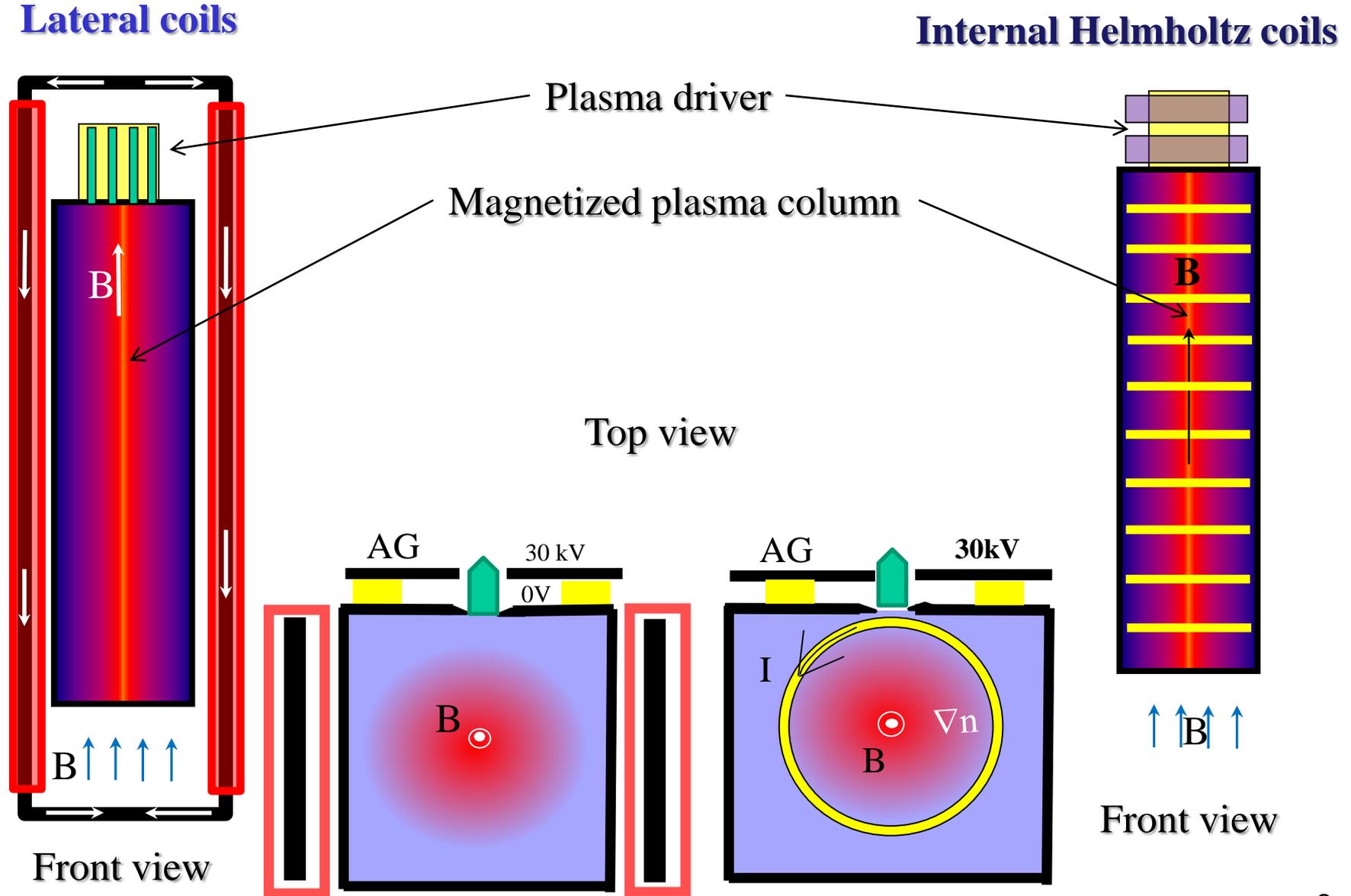


Uniform axial B-field (15-20mT), Negligible transverse B-field ( $R_{pl} \ll R_{coil}$ )  
 Very good conditions for the Helicon discharge

## Ion source concept



**Need to test the performance of Helicon antenna in another magnetic field topology than with external coils**

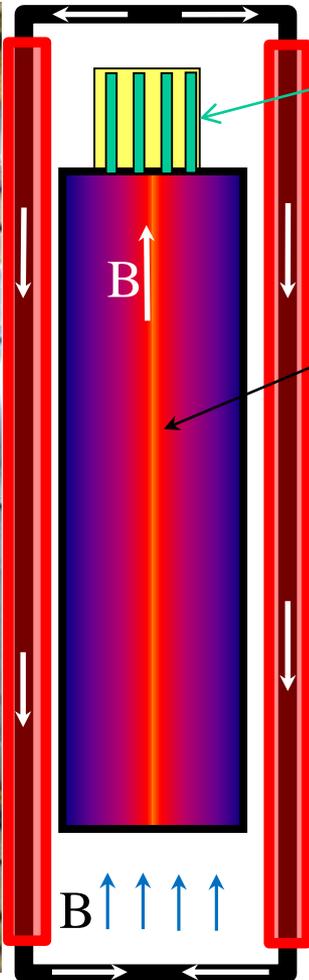


## Lateral coils

$B \sim 100 \text{ G}$

## Internal Helmholtz coils

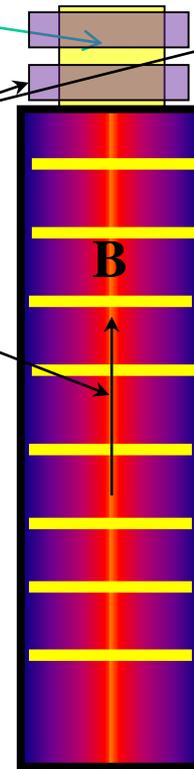
$B_{\text{axis}} \sim 100\text{-}160 \text{ G}$

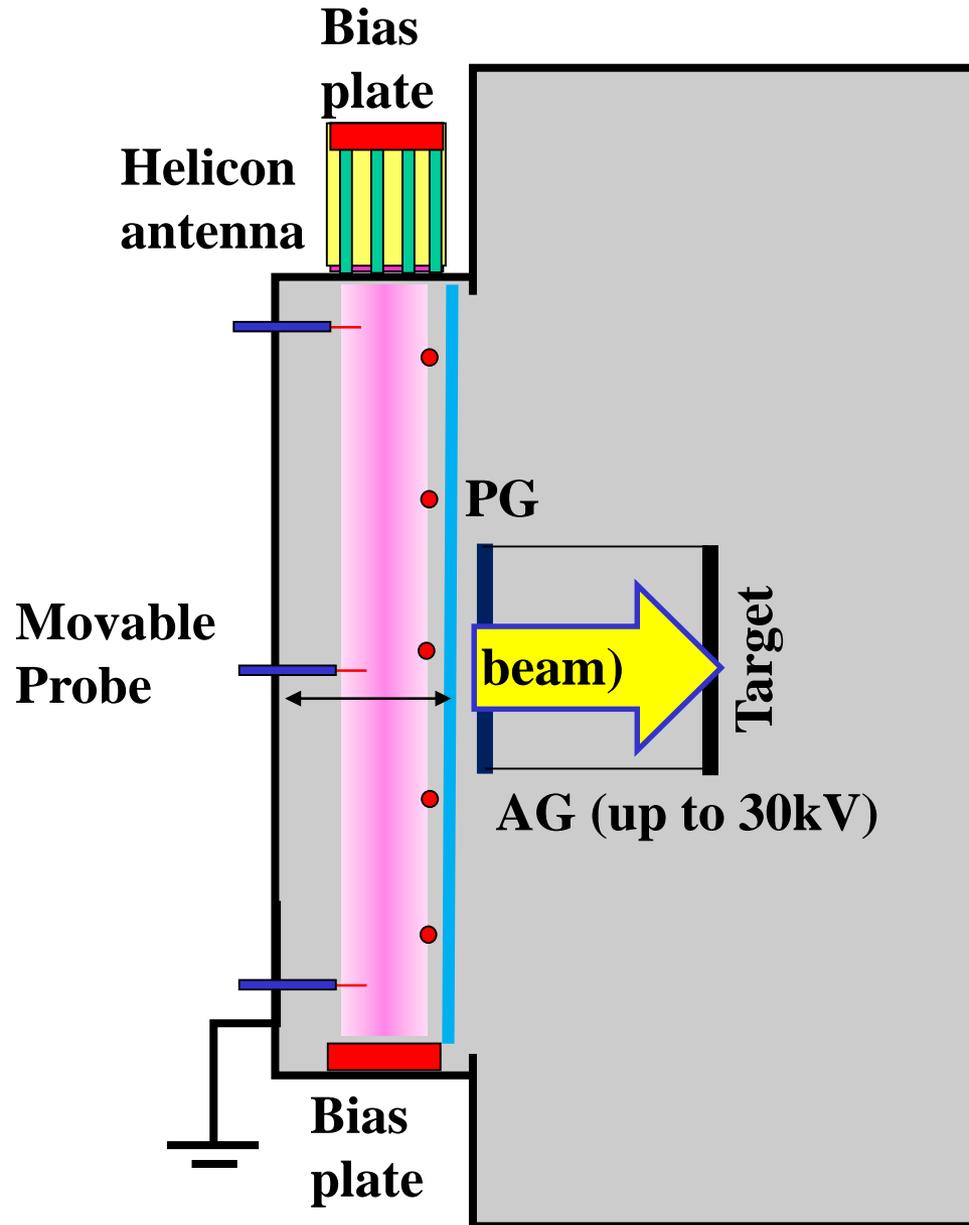


Helicon antenna

Solenoid  
around antenna

Magnetized plasma  
column





## Source side view

Experimental conditions:

Magnetic field – 100 G

RF power – 3kW

Gas pressure - ~ 0.3 Pa (H)

**Bias plates : 0V : -90V**

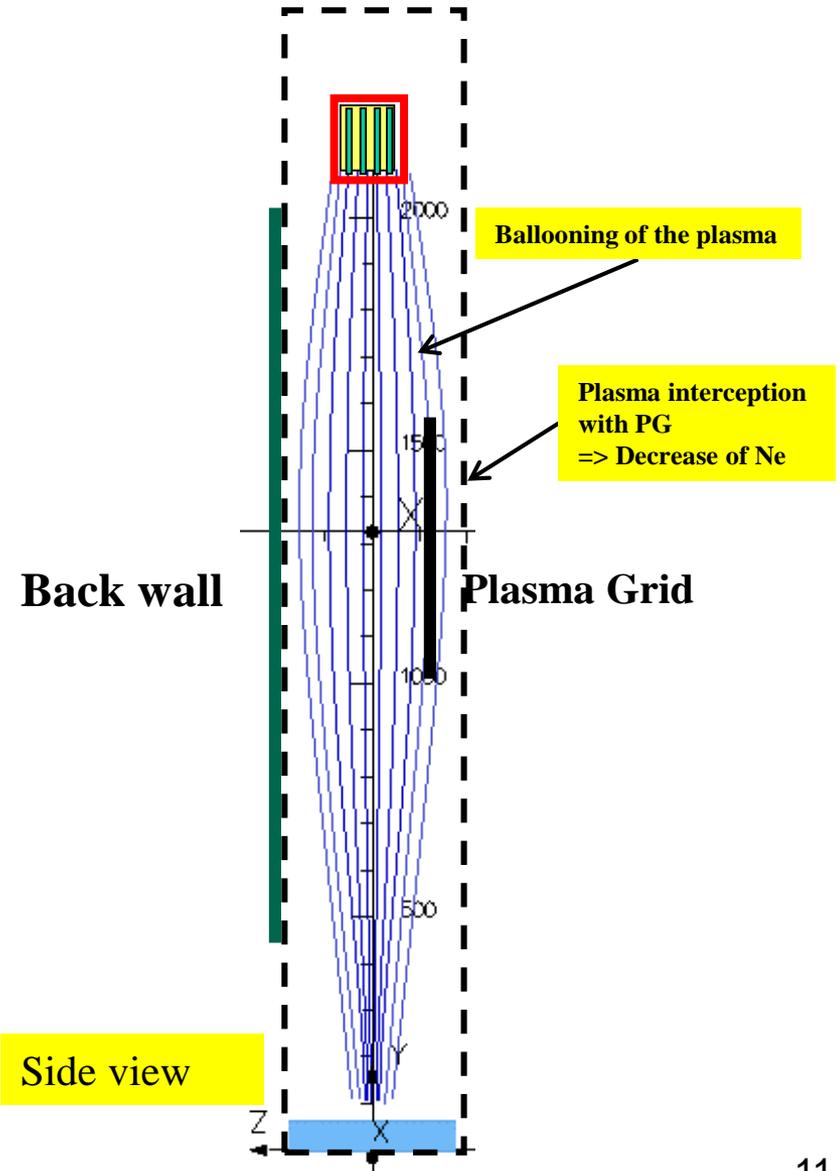
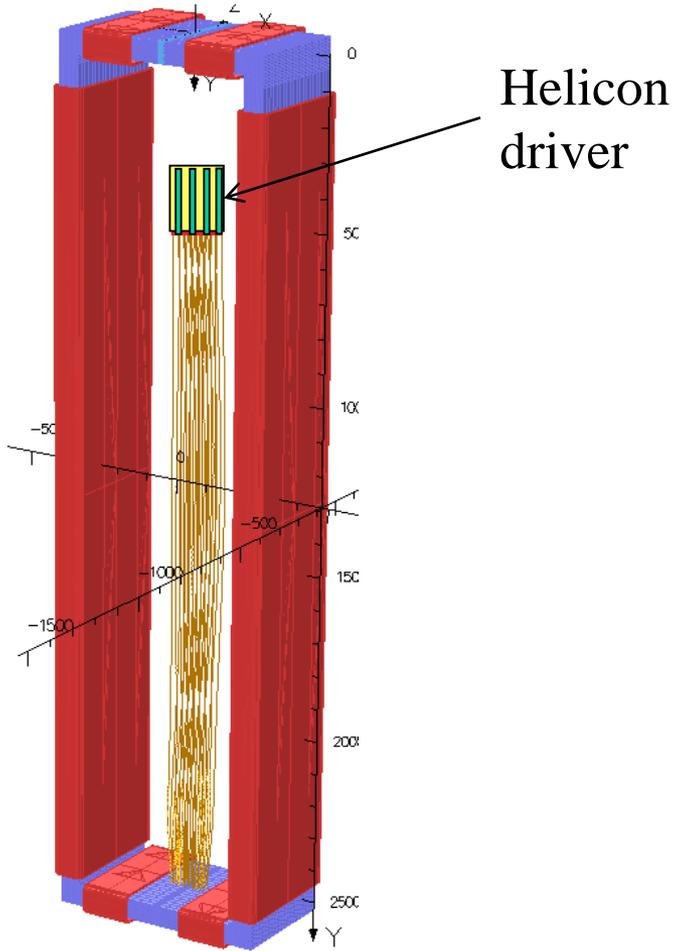
### Horizontal measurements

Movable Langmuir probe can move horizontally from the wall to the PG

### Vertical measurements

Five fixed Langmuir probes for Vertical measurements

## 3D simulations of e-trajectories (without plasma) with lateral coils



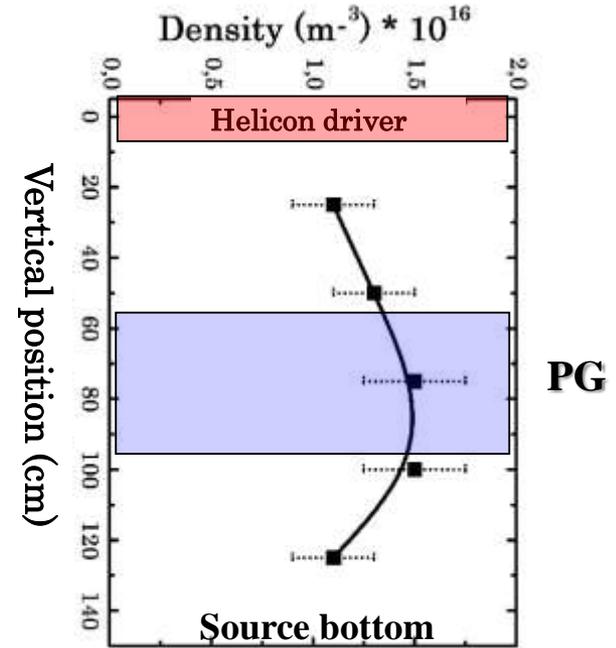
## Vertical plasma density distribution

Vertical profiles **close to PG (extraction region)**:

RF power – 3kW

Gas pressure - ~ 0.3 Pa (H)

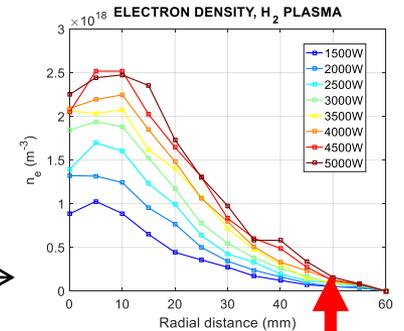
Average plasma density is  $1-1.5 \cdot 10^{16} \text{ m}^{-3}$



Lateral coils



	Filament IRFM	Helicon IRFM	Helicon EPFL RAID
Power	30kW	3kW	3kW
Ne (plasma edge)	$2 \cdot 10^{17}$	$\sim 1.2 \cdot 10^{16}$	$\sim 2 \cdot 10^{17}$



Edge plasma

WO8 Agnello, EPFL

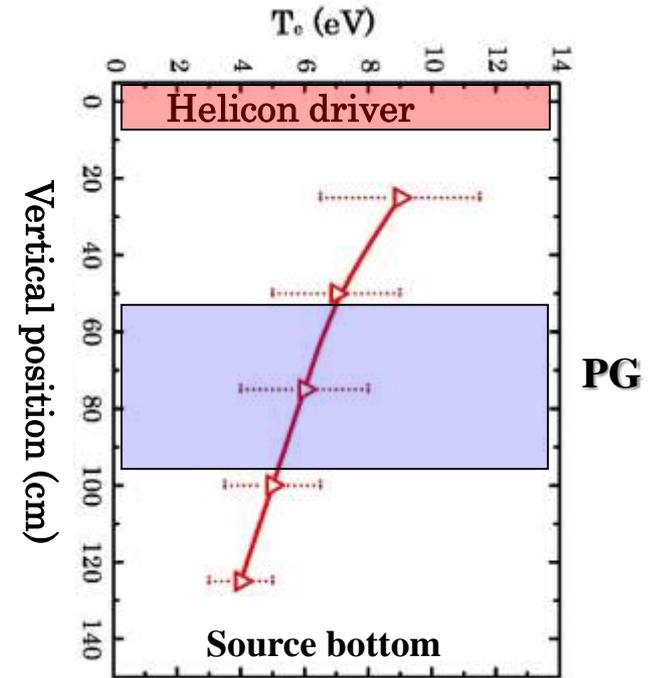
## Vertical plasma temperature distribution

Vertical profiles  $T_e$  measured close to PG:

RF power – 3kW

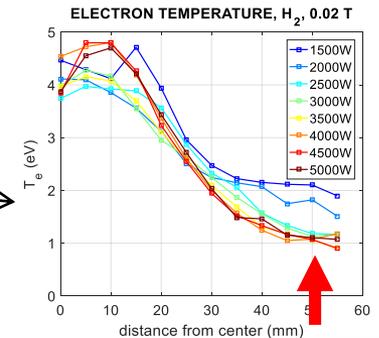
Gas pressure - ~ 0.3 Pa (H)

Temperature drops from 9 eV on the top to 4eV in the bottom



Lateral coils

	Filament IRFM	Lateral IRFM	EPFL RAID
Power	30kW	3kW	3kW
$T_e$	~4-5eV	~4-9eV	~1-2eV (50cm from driver exit)



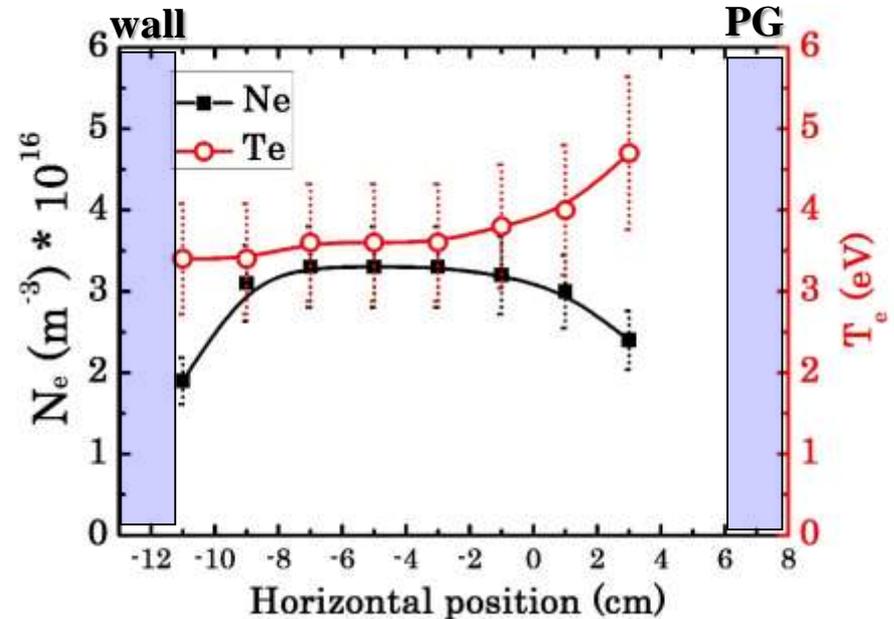
Edge plasma

WO8 Agnello, EPFL

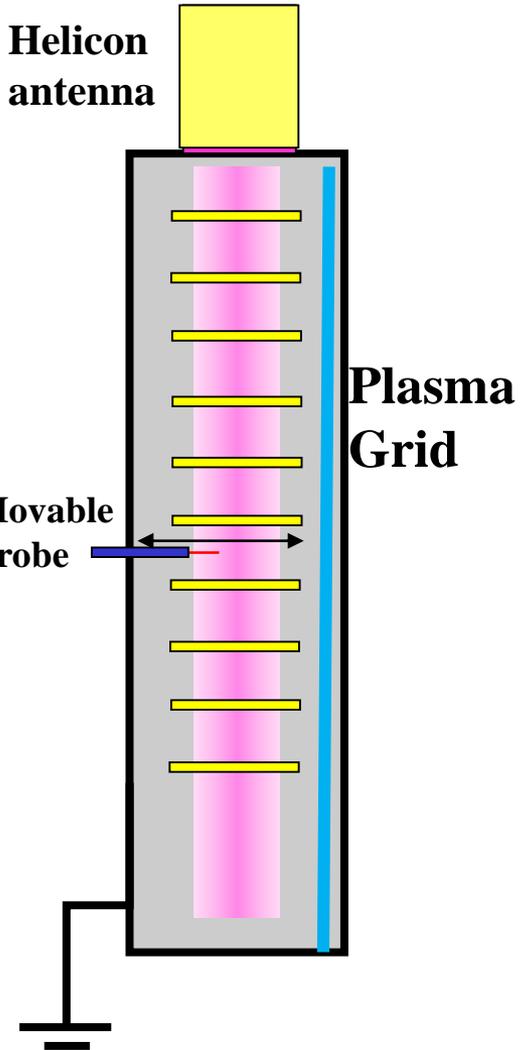
High  $T_e$  with Helicon at IRFM with lateral coils

## Horizontal plasma distribution

- 1) Low peak density  $3 \cdot 10^{16} \text{ m}^{-3}$   
(compared to  $\sim 10^{18} \text{ m}^{-3}$  at RAID EPFL)
- 2) Broad horizontal profile due to curved magnetic field lines
- 3)  $N_e$  and  $T_e$  don't have Gaussian distribution
- 4) Hot electrons on the front side of the source close to PG



**Questions : Hot e- results from plasma drift or local heating processes by interaction with the waves ???**  
**=> Need further investigations**

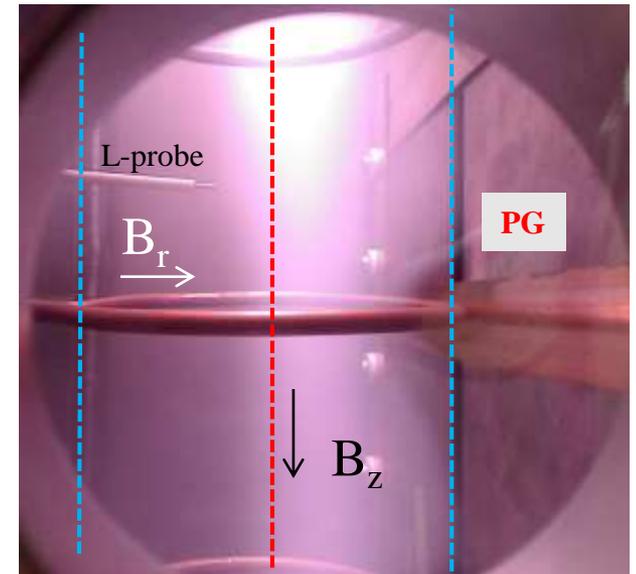


Top view

1000A ~  
B=10mT

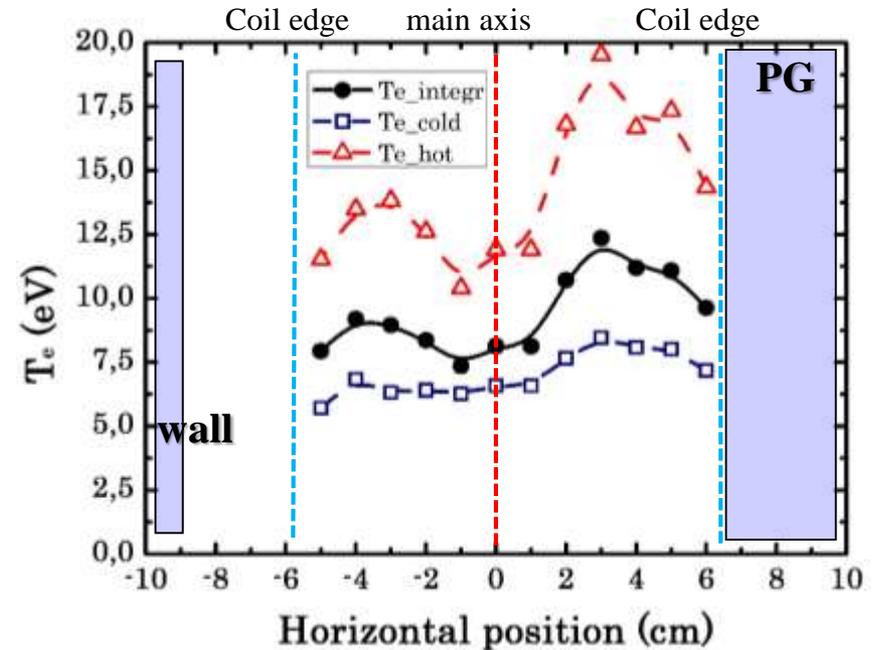
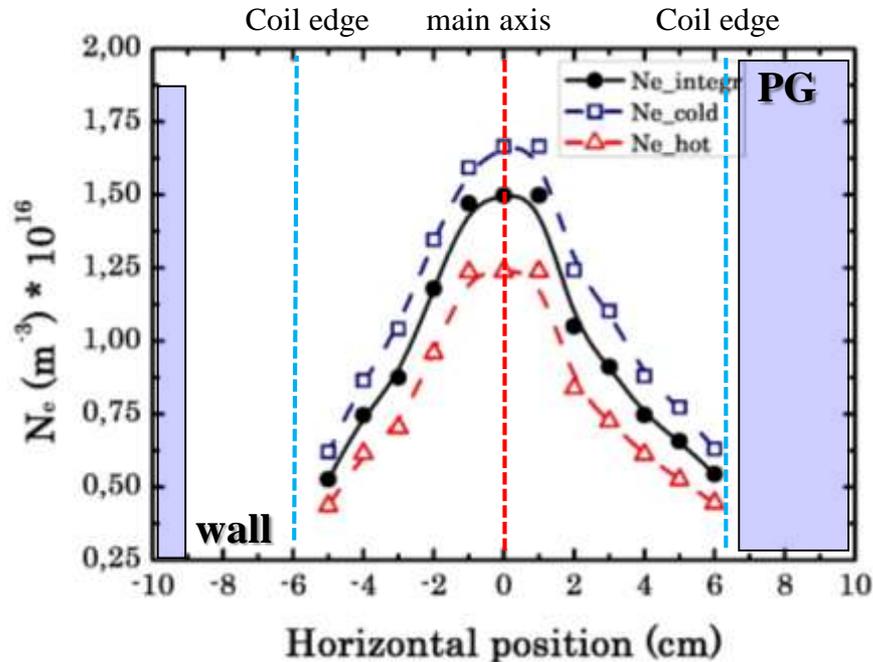


Coil edge    main axis    Coil edge



$$R_{pl} \sim 5\text{cm} \sim R_{coil} = 5,5\text{cm}$$

## Horizontal plasma distribution



-i) Plasma density is peaked (nearly Gaussian profile)

-ii) Two e- populations :

~60% of total amount of e- are “cold” with uniform distribution (~6eV)

~40% “Hot” e- are localized at the edge of plasma column (~10-20eV)

The two humps of hot e- suggest Inductive plasma generation in the antenna !!

-iii) For the same operating conditions low  $N_e \sim 1.5 \cdot 10^{16} \text{ m}^{-3}$  (compared to  $\sim 10^{18} \text{ m}^{-3}$  RAID EPFL)

Question: Does the antenna generates an ICP or Helicon wave in the column ?

=> Need to implement magnetic probes in the plasma => Collaboration with EPFL

## Wave characterization in the plasma column

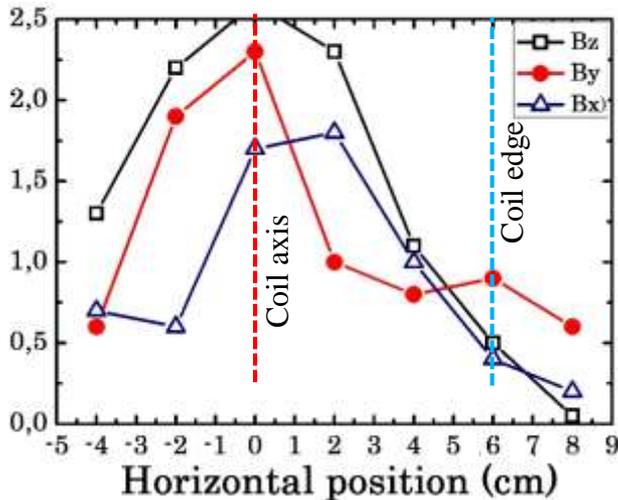
**The B-dot probe  
(provided by EPFL)**



**3 coils head.  
in the 3 axes**

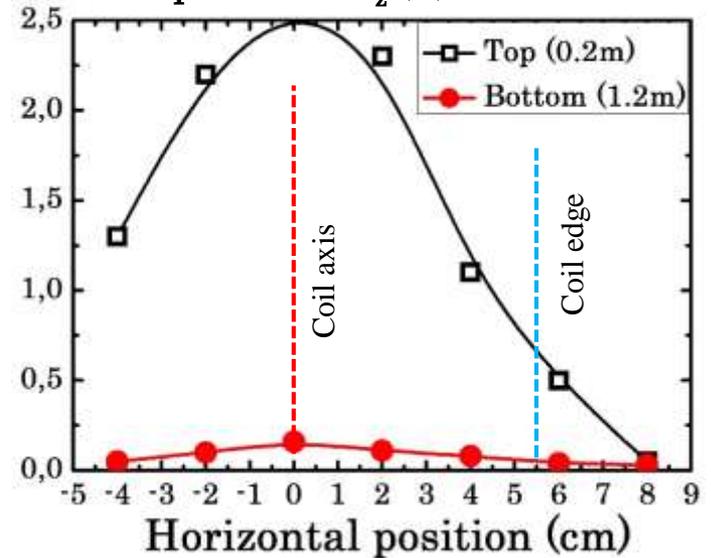


**Three components of the  
Helicon wave measured**



**Preliminary measurements indicate the presence of helicon wave in the plasma**

**Amplitude of  $B_z$  (V)**



**Damping of the helicon waves  
Along the column (Top to bottom)**

### 1) Ideal conditions in the RAID testbed for helicon plasma generation

- Wall of the vacuum tank far away from the plasma column (~20cm) compare to CEA
- Uniform axial magnetic field ~150-200 G
- Negligible transverse magnetic field  $R_{\text{plasma}} (5\text{cm}) \ll R_{\text{coils}} (25\text{cm})$

### 2) A twin Helicon antenna implemented in 2017 at CEA ion source:

- two magnetic field configurations (compatible with implementation of the accelerator are under characterization)

- the Helicon plasma column exhibit very different parameters than in EPFL:

-i) **Low Ne** for two configurations (5-10 times): plasma losses on the wall ?

-ii) **Hot e- population** at the edges close to PG (~ 7-15eV)

**”these parameters are not compatible with production of high density NI (w/wo Cs)**

Open questions (Hypothesis): Identification of this “Abnormal” e- heating process

-i) Resonant heating with Helicon waves, plasma turbulence, ???

-ii) Interaction of the wave with the horizontal component of B ???

-) Low Hybrid resonance?  $f_{\text{LH}} \sim 10 - 50 \text{ MHz}$  (helicon antenna at 13,56MHz)

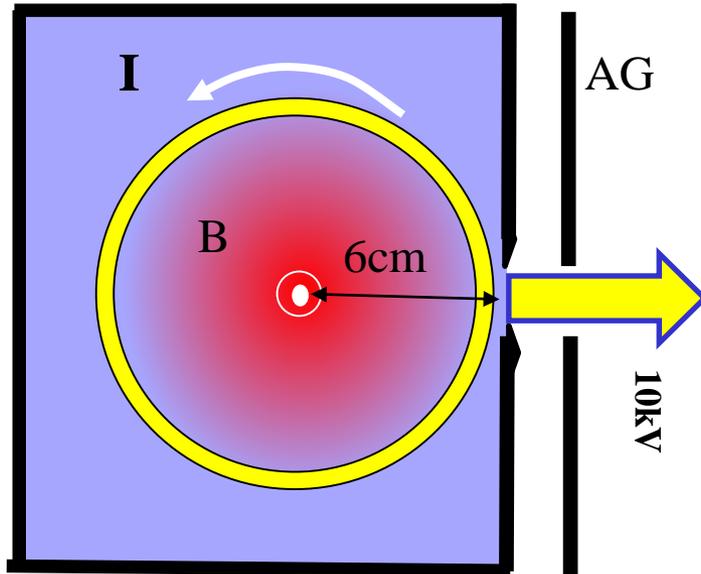
-) Alfvén waves ? Alfvén wave velocity same order than electron velocity

=> Electron heating by Landau damping

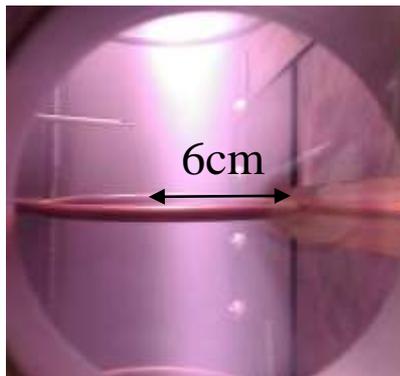
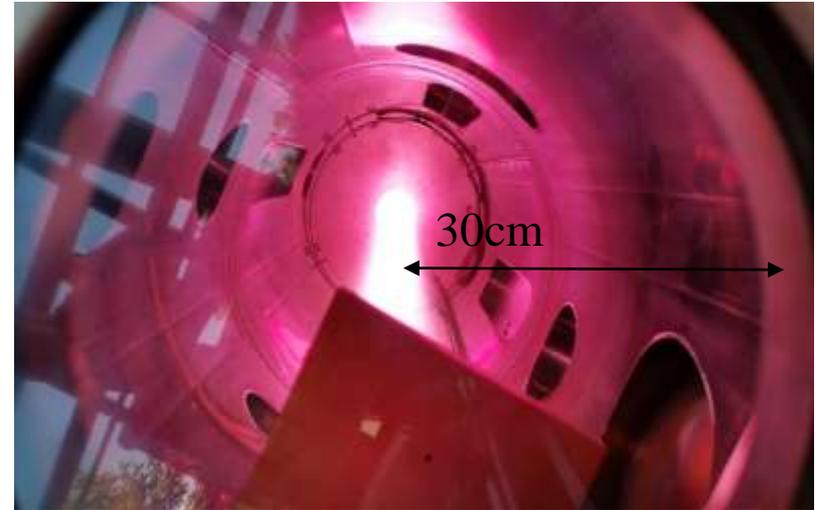
→ Further investigations are required

**Thank You  
for attention!**

## CEA Cadarache

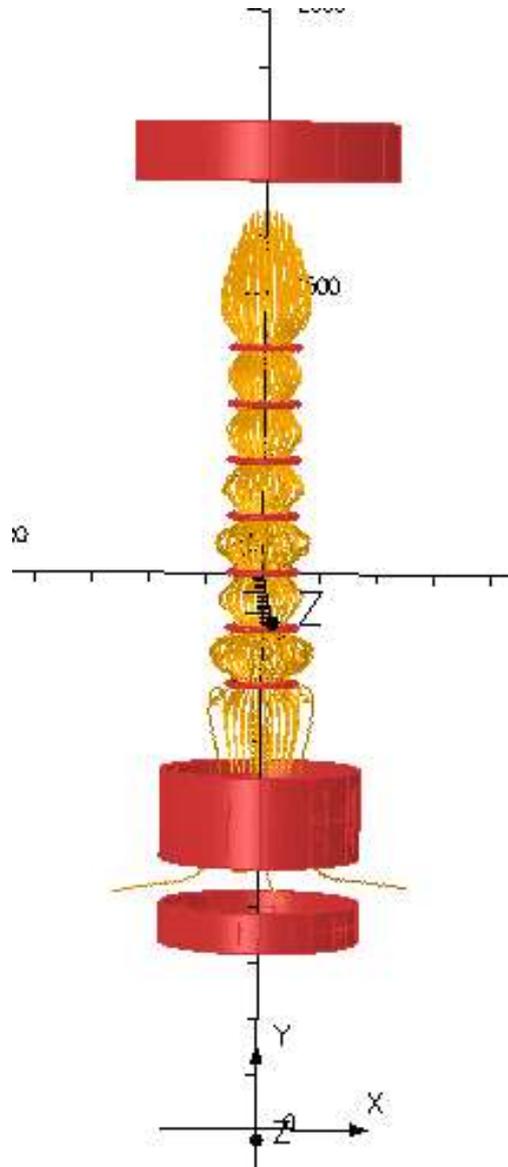


## EPFL

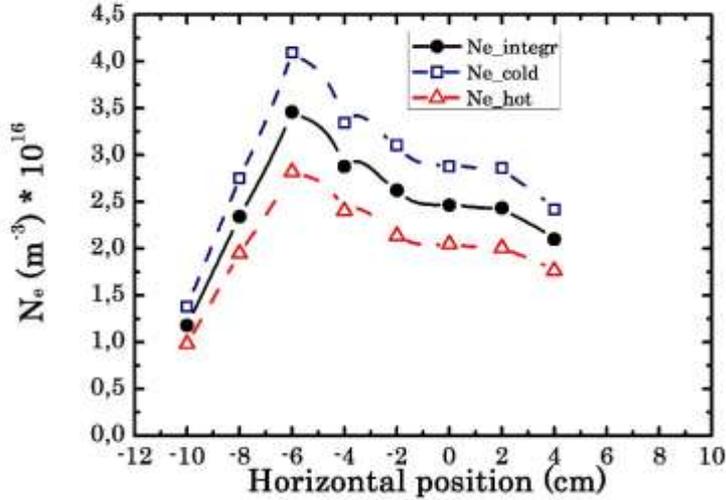


**Effect of the big conduction plate at the distance 6 cm from the plasma column has to be checked**

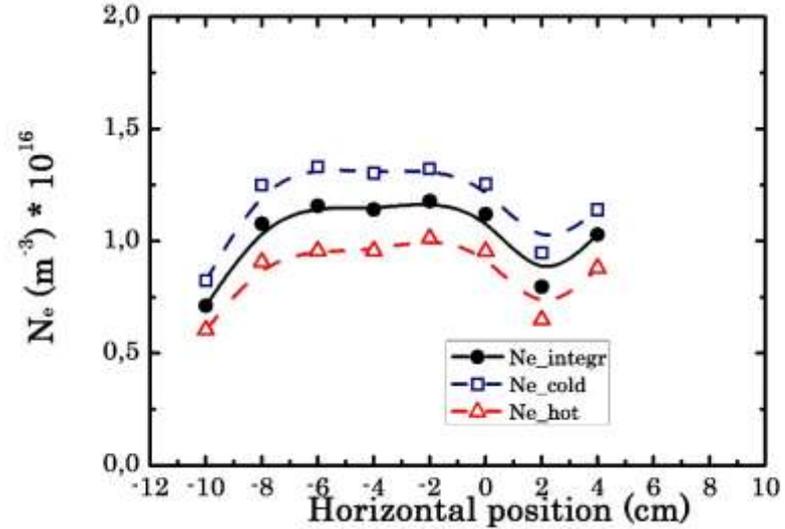
## 3D simulations of e-trajectories (without plasma)



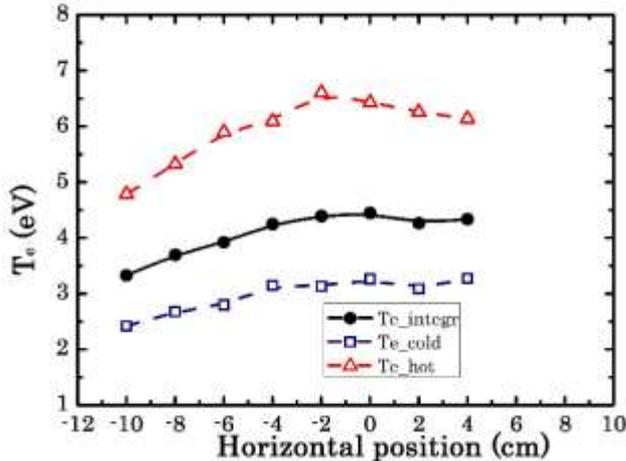
Ne, sh1444, float plates ,no\_rot, Varc=32V, chm1=200, chm2=126, lateral



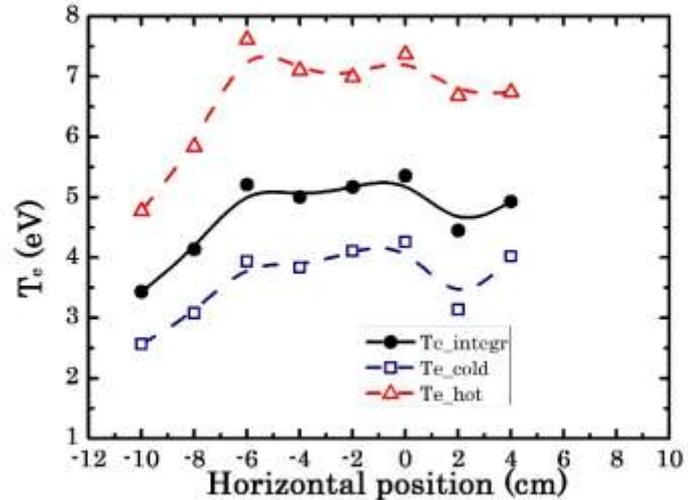
Ne, sh1461, ground plates ,no\_rot, Varc=32V, chm1=200, chm2=126, lateral

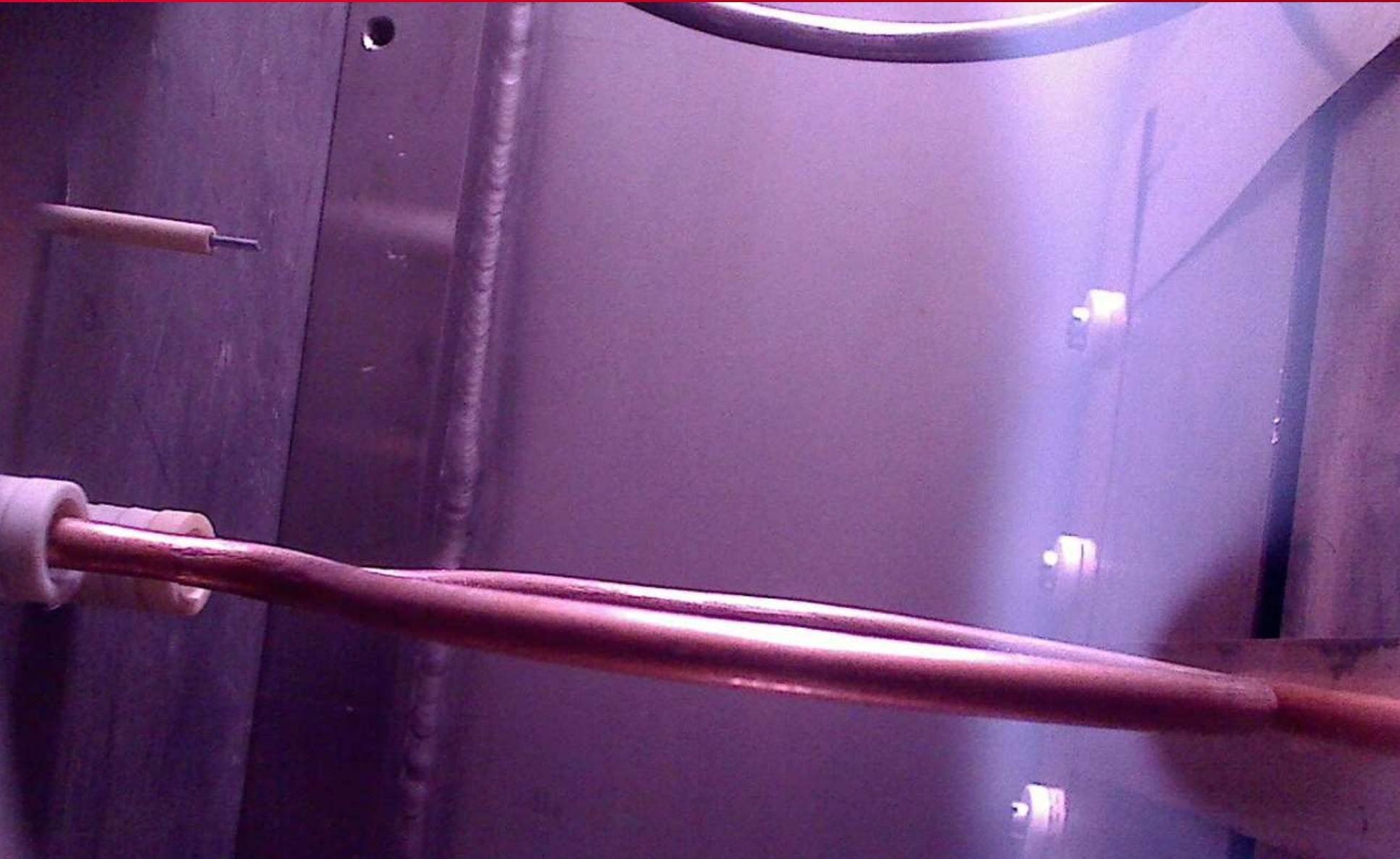


Te, sh1444, float plates ,no\_rot, Varc=32V, chm1=200, chm2=126, lateral



Te, sh1461, ground plates ,no\_rot, Varc=32V, chm1=200, chm2=126, lateral

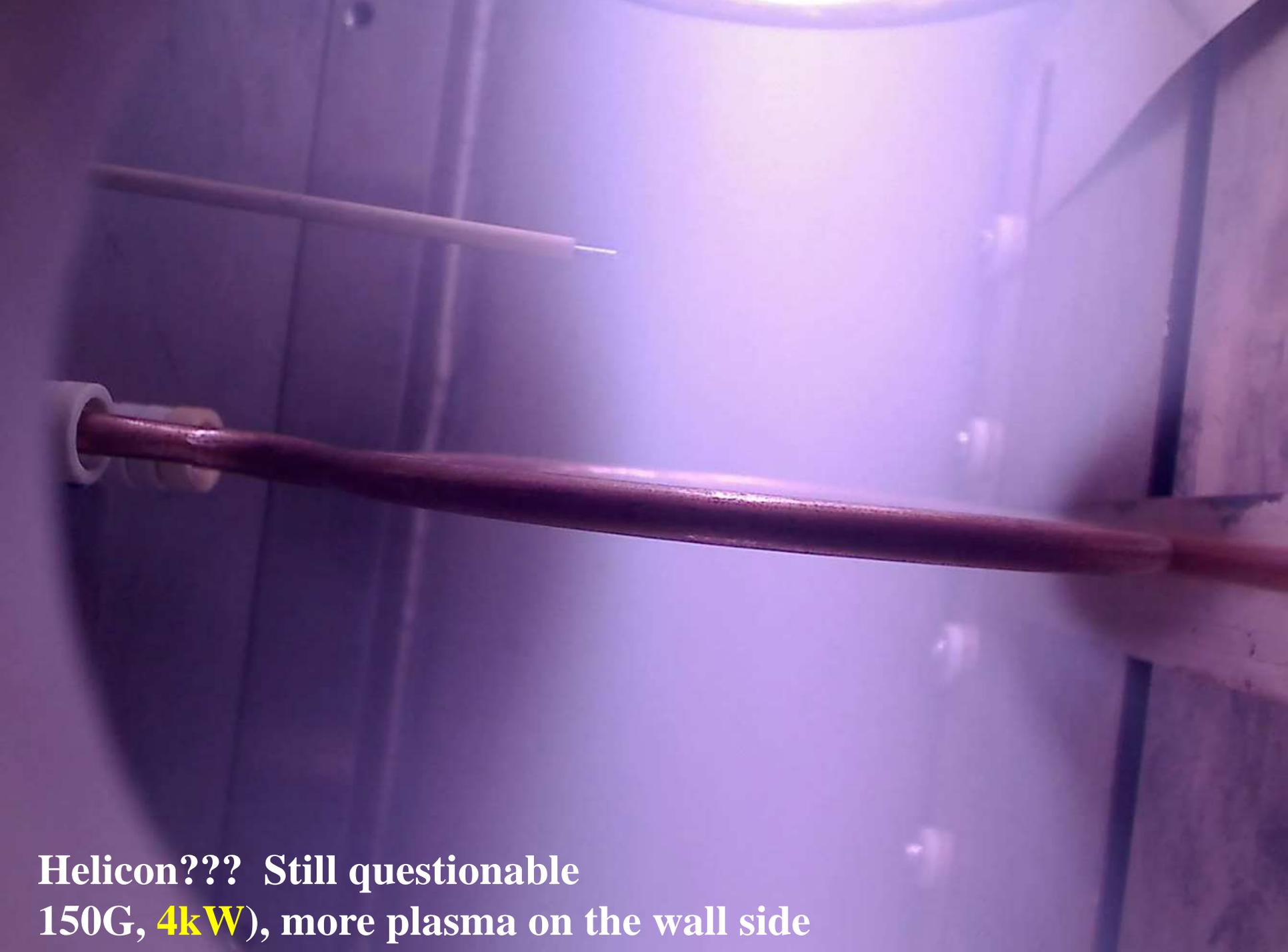




**Transition from ICP to Helicon???**  
**Low B filed (80-90 G, 3kW)**



Helicon??? Still questionable  
150G, **3kW**), less plasma on the wall side compare to PG



Helicon??? Still questionable  
150G, 4kW), more plasma on the wall side



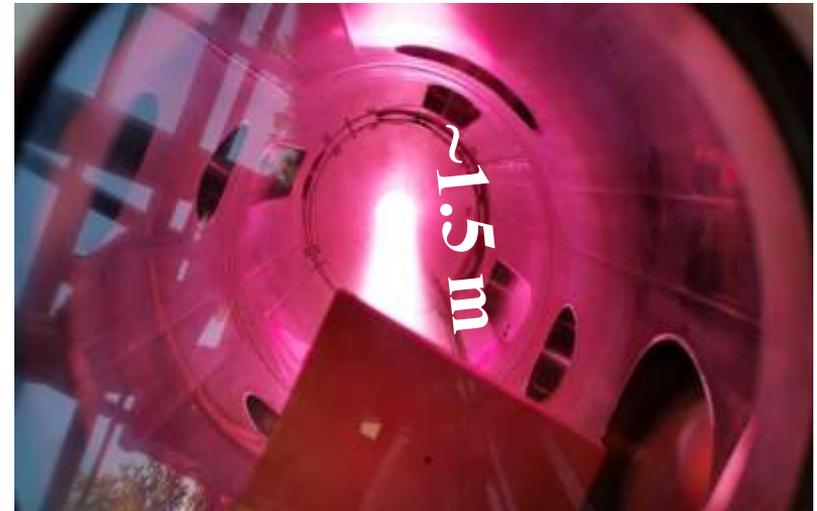
Helicon??? Still questionable  
150G, **5kW**), again more plasma on the wall side

Development of a 10 kW Helicon antenna (Bird-cage type) at RAID testbed (EPFL) to provide a dense magnetized plasma column

3 kW, 0.3 Pa, B= 12 mT, H<sub>2</sub> plasma jet

Helicon antenna is essential for creation of the homogeneous plasma column

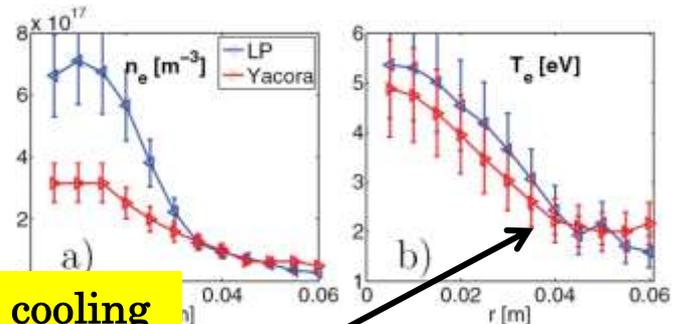
- Low B-field (~10 mT)
- ITER relevant operating pressure (~0.2 Pa)



**But no NI extraction possible in such topology**

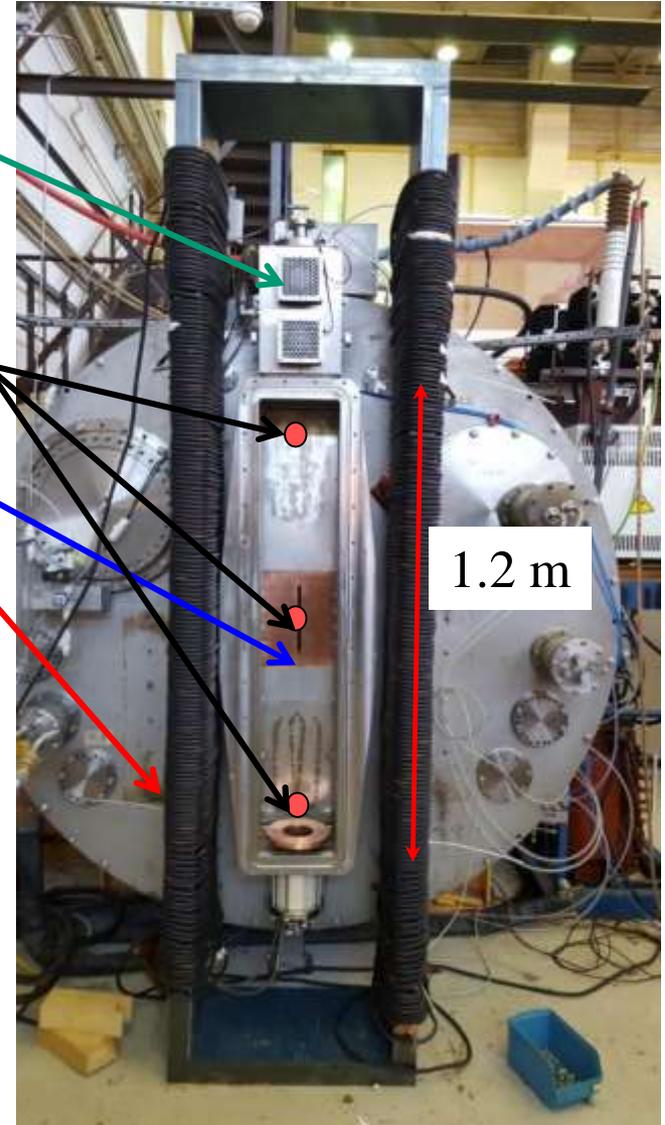
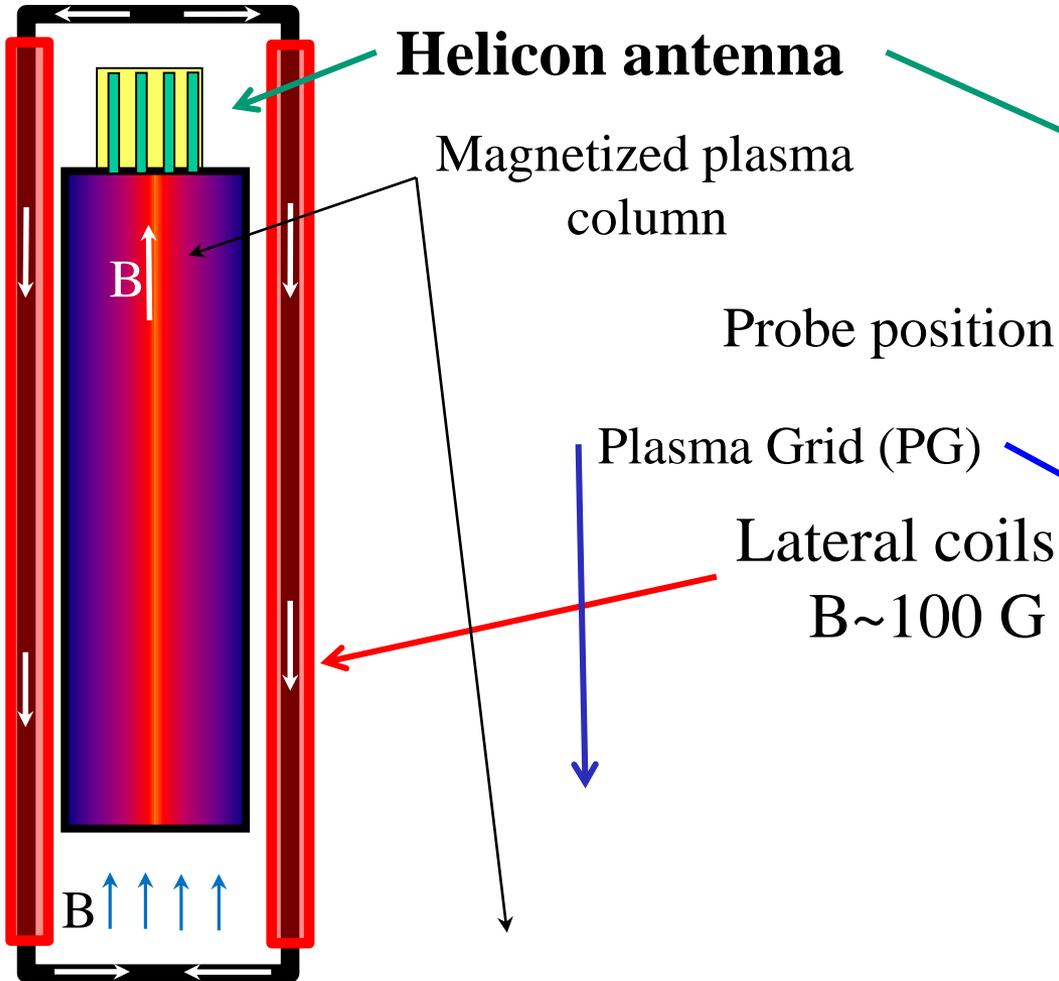
The **GOAL** at CEA IRFM is to get similar results as in EPFL [4]

With another magnetic topology



Radial cooling (T<sub>e</sub> < 2eV)

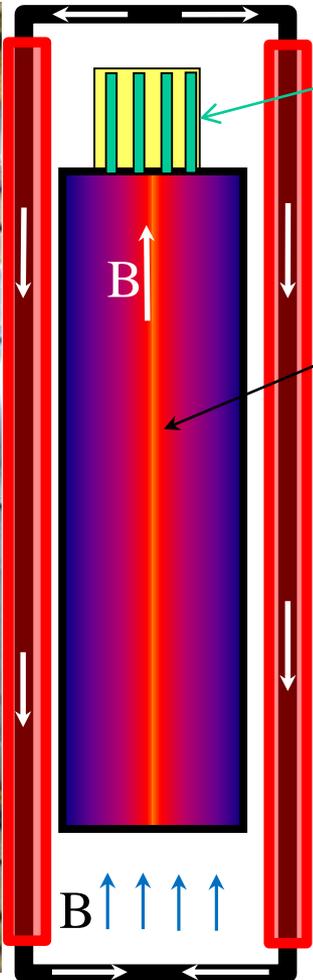
## Lateral coils (configuration 2017)



## Lateral coils

(**OLD configuration**)

$B \sim 100$  G



## Set of 11 vertical coils

(**NEW configuration**)

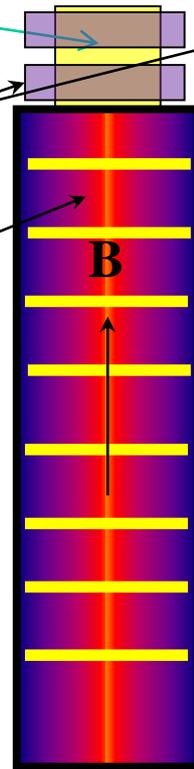
Baxis  $\sim 100-160$  G

Helicon antenna

Solenoid  
around antenna

Magnetized plasma  
column

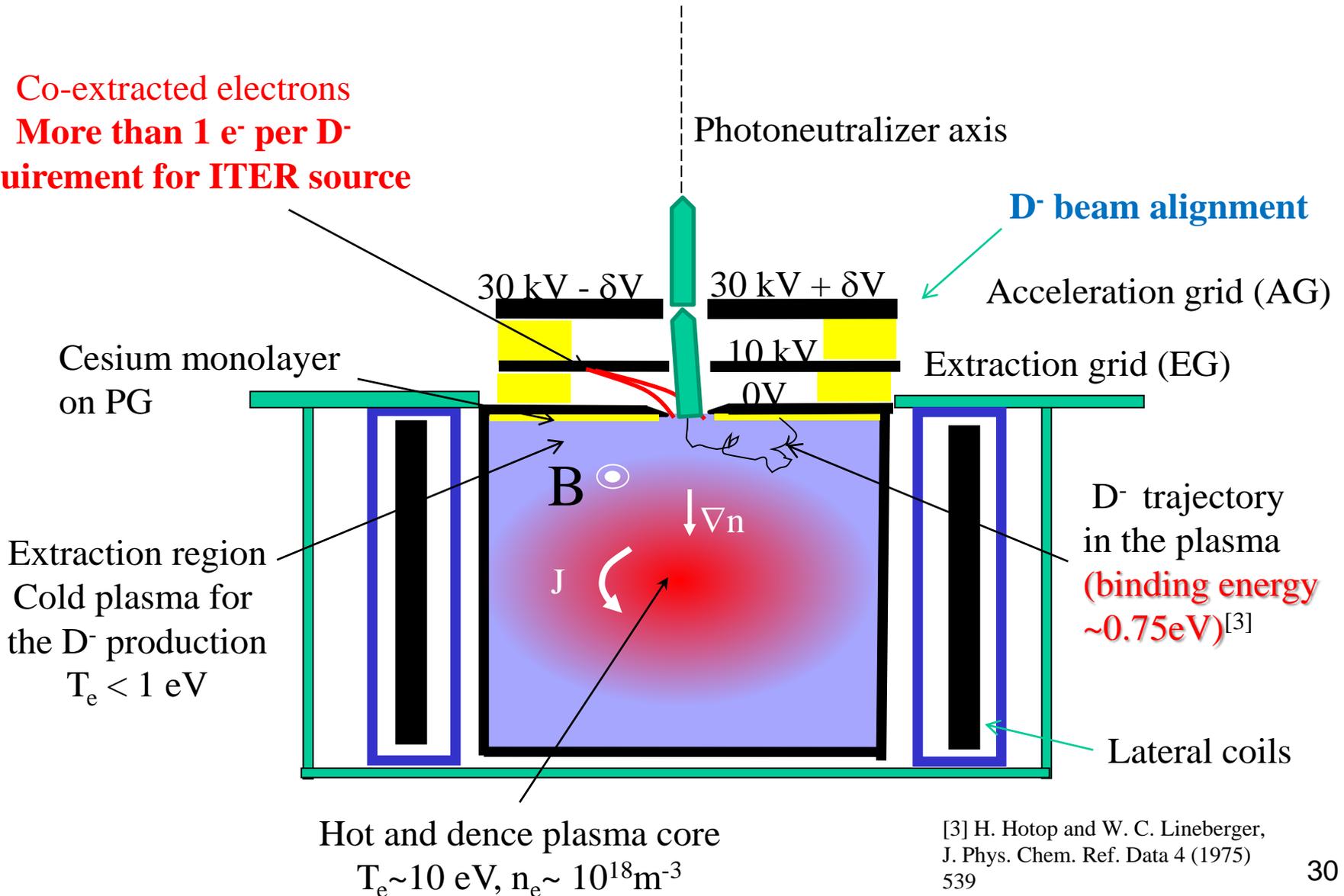
Helmholtz coils



Experiments started in March 2018

## Ion Source with 30 kV acceleration

Co-extracted electrons  
**More than 1 e<sup>-</sup> per D<sup>-</sup>**  
**requirement for ITER source**

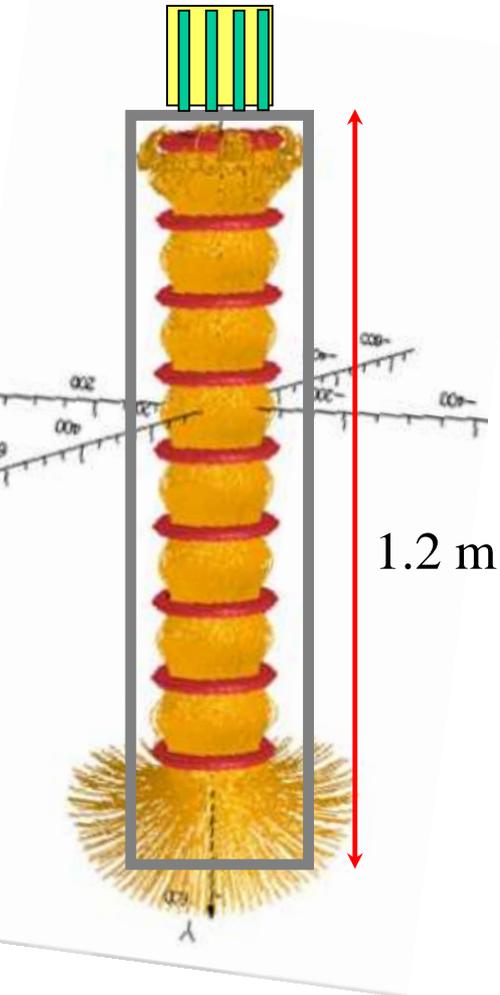


[3] H. Hotop and W. C. Lineberger, J. Phys. Chem. Ref. Data 4 (1975) 539

- 1) Two specific magnetic field configurations (Lateral coil and set of Helmholtz coils) were tested.
- 2) The Langmuir probes measurements have highlighted a plasma asymmetry between the back and front side (PG) of the source , a dense plasma core is shifted to the back wall of vacuum chamber, while on PG, the plasma is hotter (6-7eV) due to the primary electron drift – not favorable for production of NI.
- 3) A new magnetic configuration composed of Helmholtz coils implanted within the source vacuum chamber , tested and characterized in 2018.
- 4) The Langmuir probes measurements revealed also a plasma asymmetry between the back and front side (PG) of the source , a dense plasma core is shifted to PG, while on PG, the plasma is hotter – not favorable for production of NI.
- 5) New set of experiments will be performed in August for detection of the Helicon wave propagation inside the source volume.
- 6) Need to compare the results with EPFL with implementation of the metal plate close to the plasma border.

- 1) The lateral coil configuration requires the perfect transverse (or lateral) alignment of the Helicon antenna with vertical magnetic field (sensitivity is few mm range)
- 2) With the perfect alignment we have slab plasma shape. It is can be the advantage for the NI extraction in the future.
- 3) The small misalignment induce a strong drift of e- and inhomogeneous plasma in the transverse direction.
- 4) Cylindrical plasma column (EPFL type) can be obtained in with the implementation of solenoidal coil --> the program for the 2018 (Characterization of the plasma in this magnetic field topology)

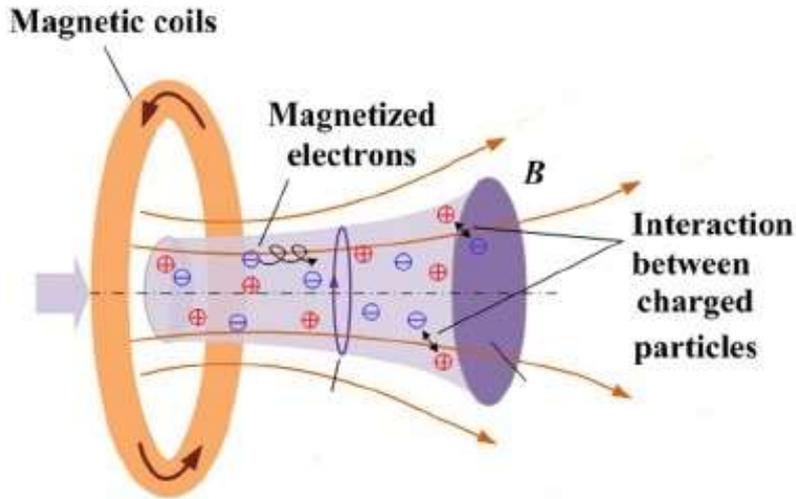
## Simulations of the 3D e-trajectories in the set of Helmholtz coils



Plasma column produced by the Helicon driver  
surrounded by **Set of 9 coils inside vacuum chamber**  
 $B_{\text{vert}} = 110$  Gauss

A new magnetic field configuration is under  
development for Cybele.

Start of the experiment in February 2018

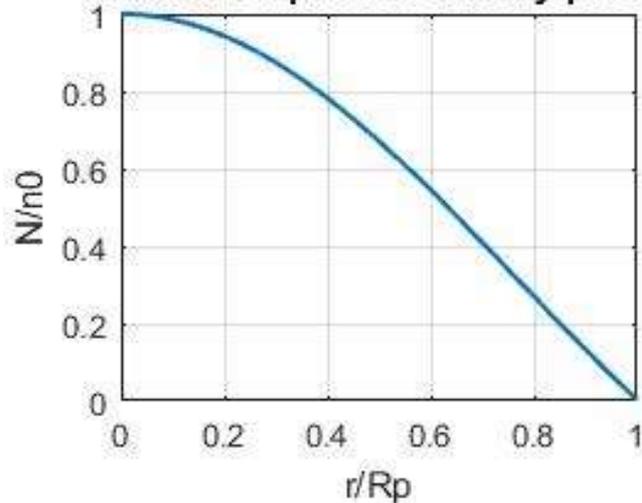


Low temperature ( $T_e \sim 10\text{eV}$ ), and weakly magnetized ( $B=100\text{-}500\text{ G}$ ) plasma

$e^-$  magnetized (move mainly along MF lines)

$i^+$  unmagnetized (can move across MF lines). Ambipolar  $E_r$  builds to confine  $i^+$

Normalized plasma density profile

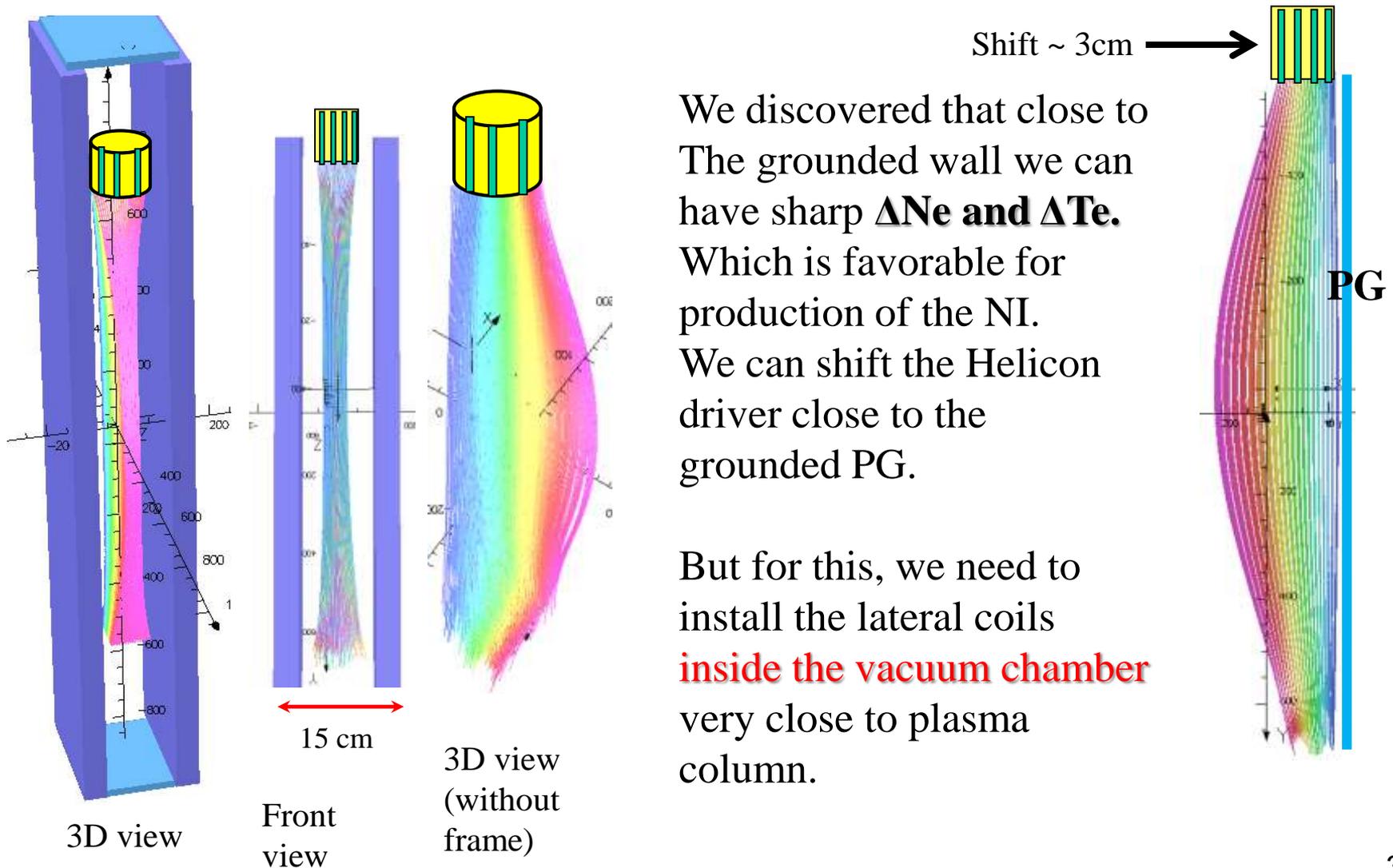


Axial MF creates:

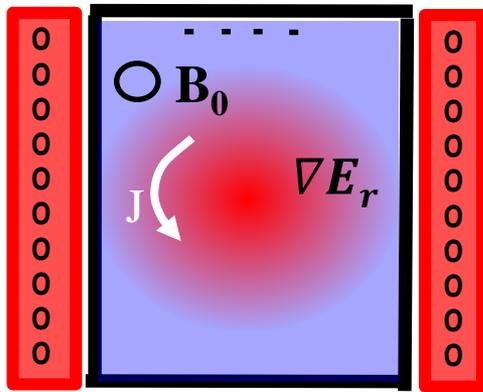
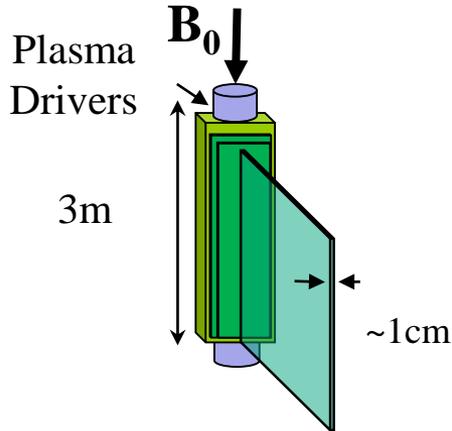
- 1) Sharp  $\nabla n(r)$   $\nabla T(r)$
- 2) Hot and dense plasma core, lower density and colder plasma edge
- 3) Ambipolar  $E_r$  due to ion diffusion ( $D_i$ ) across MF
- 4) Different types of instabilities (rotational, ExB drift, diamagnetic drift, )

$$n(r) = n_0 J_0 \left( \frac{\alpha r}{R_{pl}} \right), \alpha = f(\mu_i, \mu_e, \nu_{ei}, D_i, D_e)$$

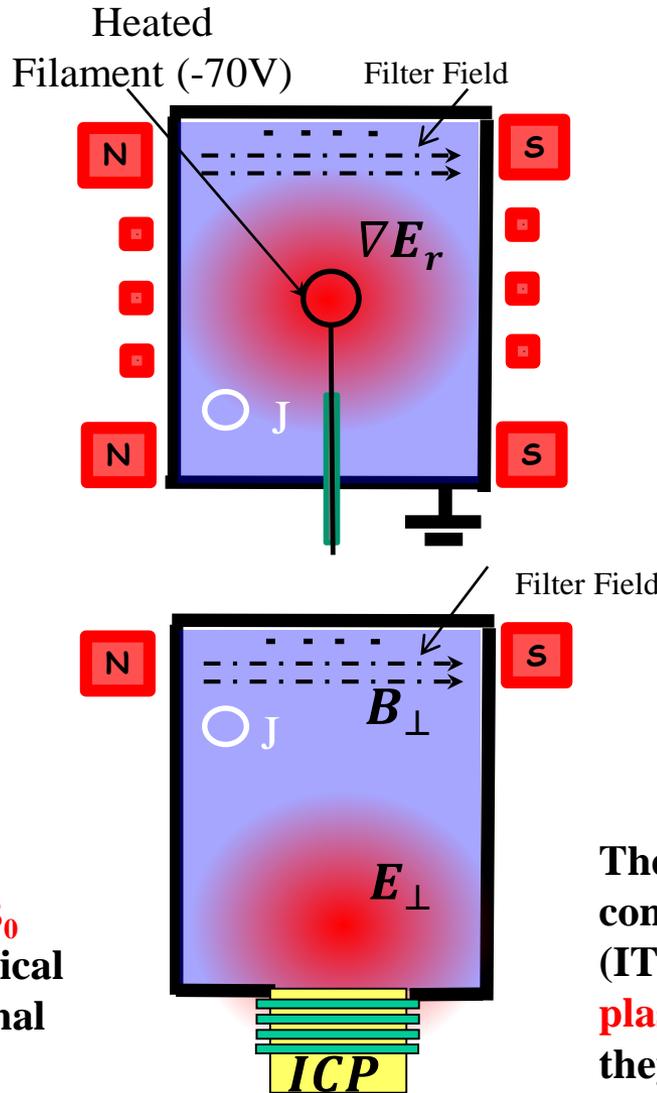
## Simulations of the 3D e-trajectories in the small lateral coils Implemented inside the source volume under vacuum



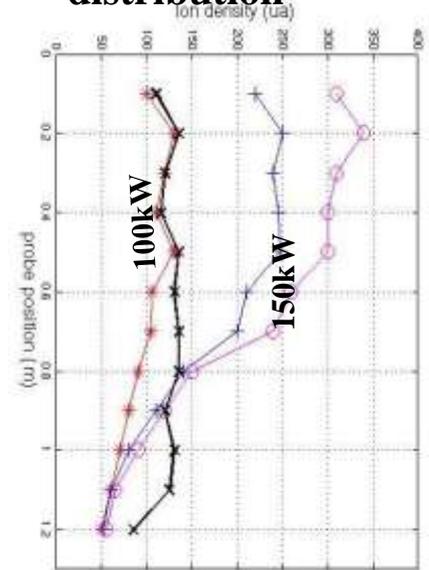
The vertical inhomogeneity of the beam required  $< 10\%$



Magnetized plasma column **along  $B_0$**   
Plasma is homogeneous along the vertical axis  
 $E \times B$  drift can cause only azimuthal plasma rotation



Vertical plasma distribution



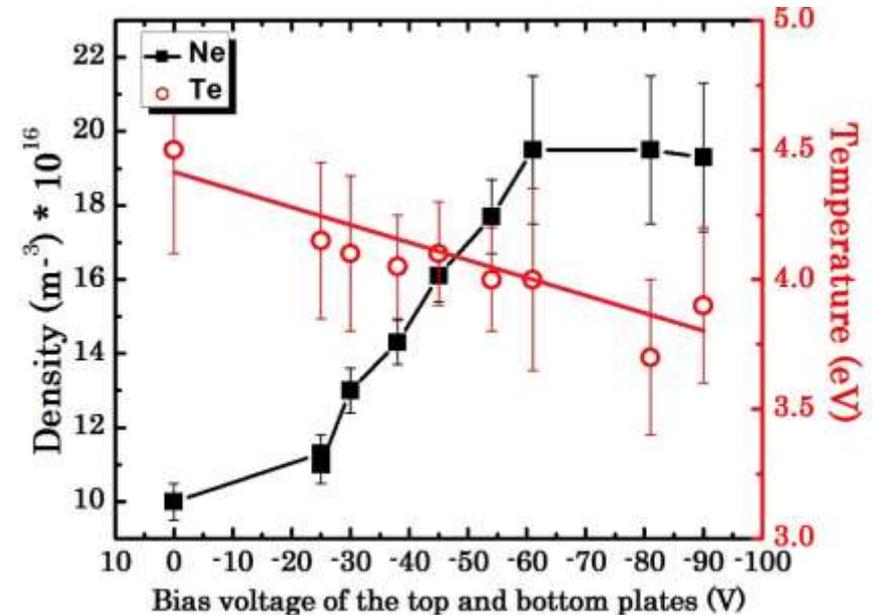
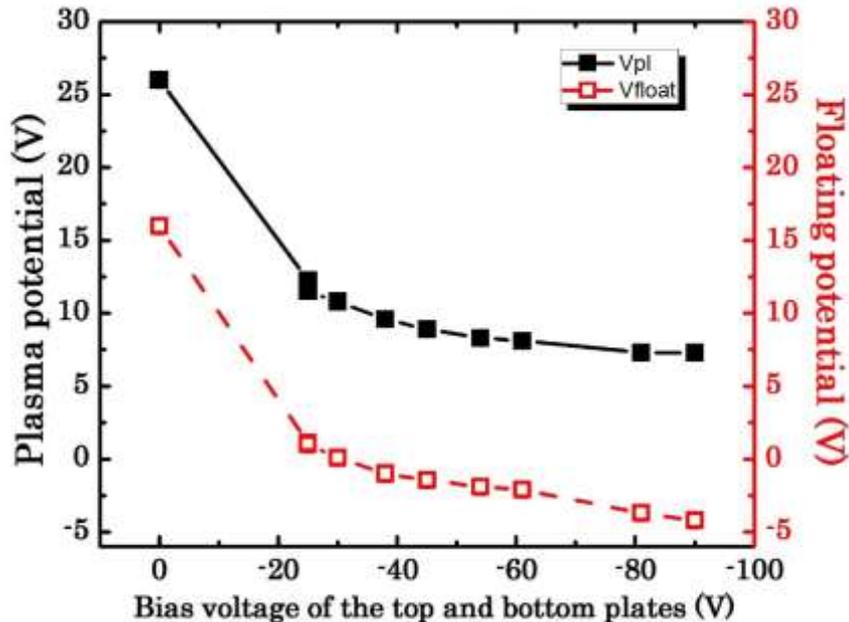
strong plasma inhomogeneity along vertical axis

The main problem of conventional ion sources (ITER like in IPP Garching) is **plasma vertical drift** which they can not overcome

Plasma particles injected along B field

## Effect of the bias on the top and bottom plates

Variation of the top and bottom bias potential  $\sim 4$  cm from the back wall



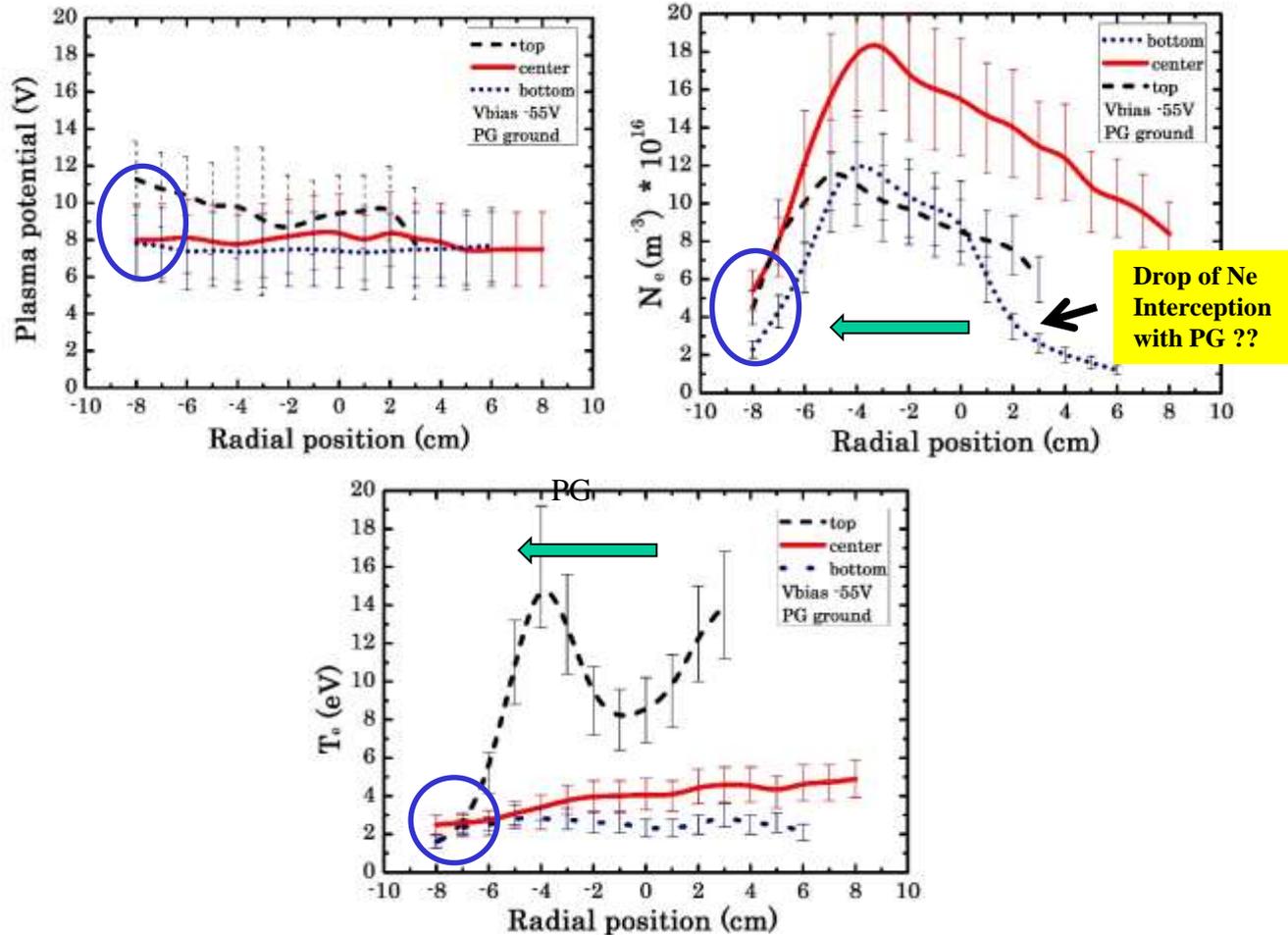
With increasing of the negative bias

- 1) Both Vpl and Vfl drops by  $\sim 15$ V
- 2) Ne increases until -60V of bias, after that constant
- 3) Te linearly decreasing from  $\sim 4.5$  to  $\sim 3.5$ eV

# Plasma distribution from top to bottom

## Transverse distribution of plasma parameters

-----Top 10cm, center 60cm, .....bottom 110 cm from the exit of Helicon driver



- 1)  $V_{pl}$  is nearly the same in the all source volume
- 2)  $V_{fl}$  drops towards the bottom
- 3) Ne is almost twice higher on the center (close to PG) with respect to extremities
- 4)  $T_e$  has two maximums at -4cm and close to PG at the top.

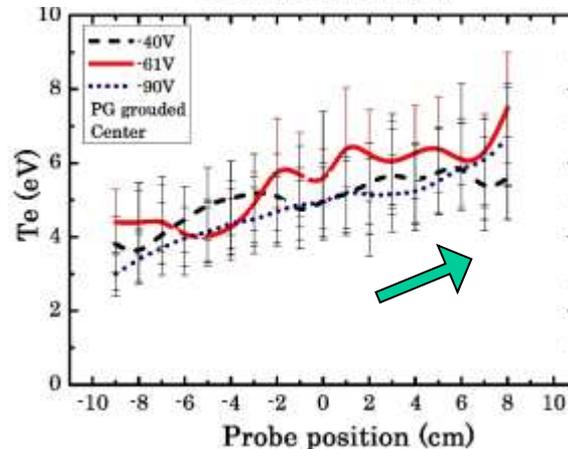
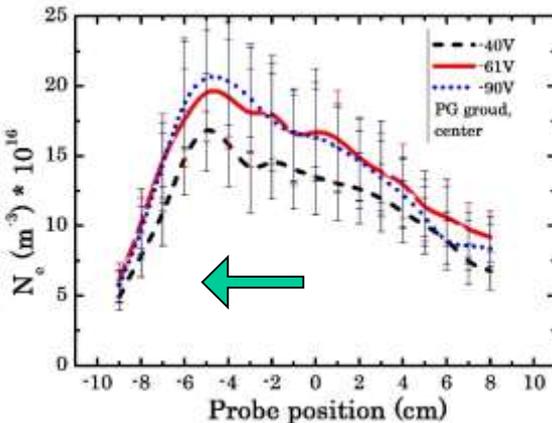
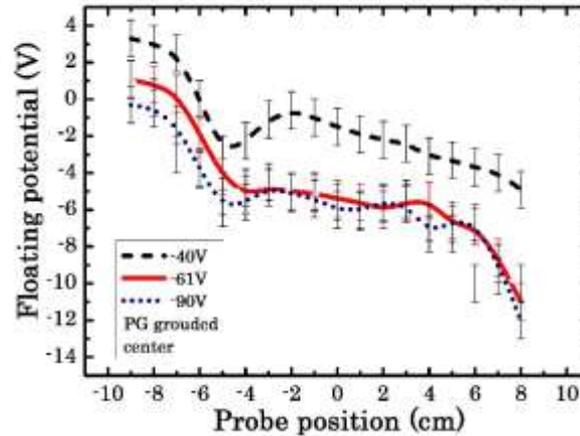
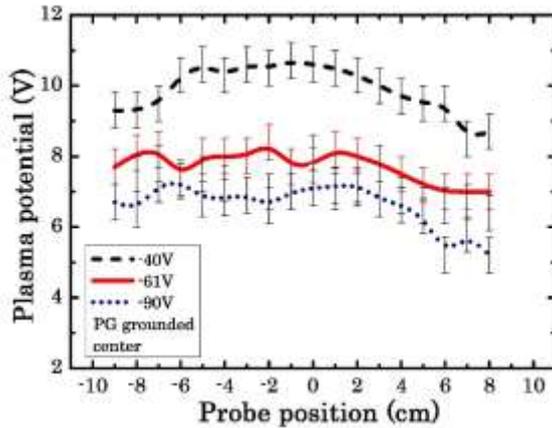
There are two plasma electron populations:  
 First closer to wall at -5-4cm with high density  
 Second is low density e-population which drifts close to PG

(PG grounded, V bias -47V, Prf 3kW, p=0.3 Pa, B=100G, H).

# Effect of the bias on the top and bottom plates

Transverse distribution of plasma parameters

Probe location: centre position (behind the PG)



Increasing of Negative bias

- 1)  $V_{pl}$  drops
- 2)  $V_{fl}$  drops
- 3)  $N_e$  increases
- 4) No effect on  $T_e$

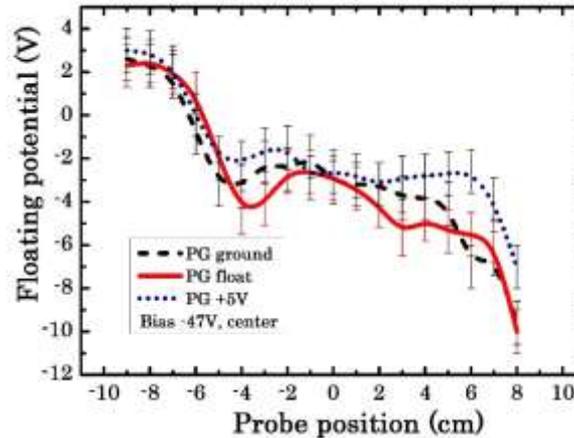
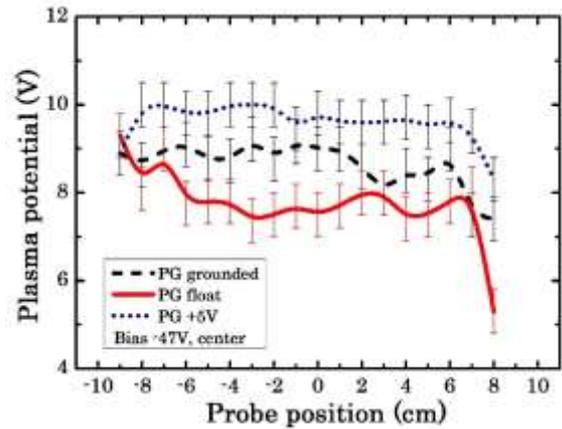
Behavior of profiles from wall to PG

- 1)  $V_{pl}$  is almost constant
- 2)  $V_{fl}$  drops toward PG by 7-10V
- 3)  $N_e$  has maximum at -5cm
- 4)  $T_e$  increases from 4 to 6 eV

(60 cm from the Helicon driver, PG is grounded, Prf 3kW, p=0.3 Pa, B=100G, H).

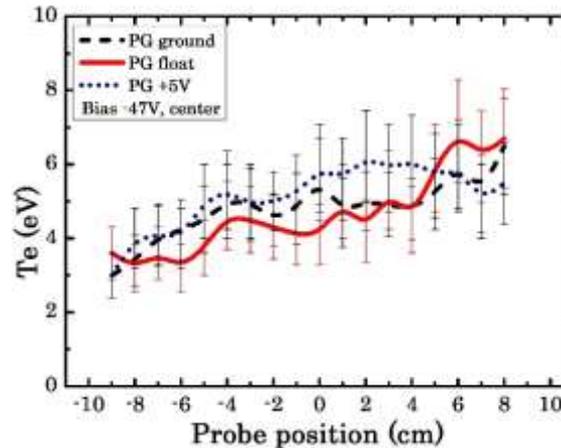
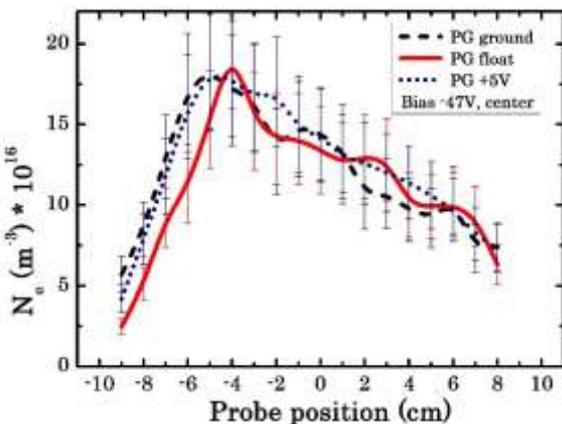
# Effect of The Plasma Grid polarization

Transverse distribution of plasma parameters  
 Probe location: centre position behind the PG

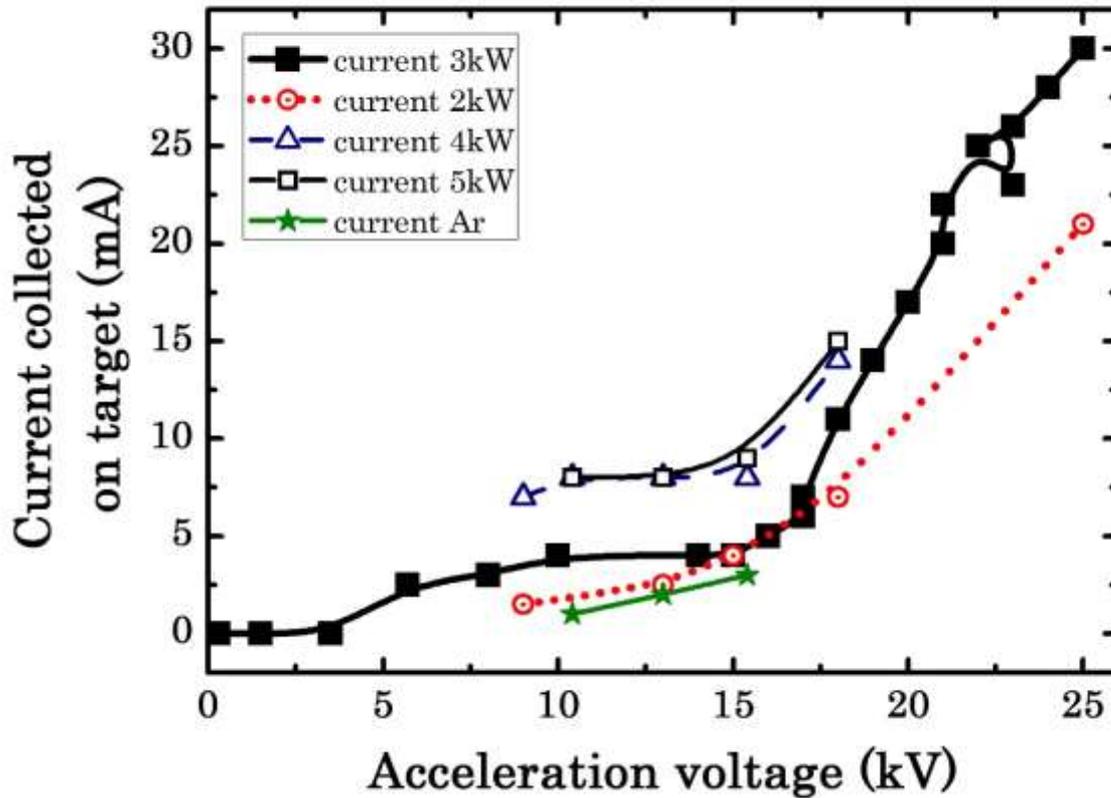


- 1)  $V_{pl}$  drops close to PG by 2-3V
- 2)  $V_{fl}$  drops towards PG by 8-10V, no effect from PG polarization
- 3) Ne has maximum at -5
- 4) No effect on  $T_e$ , increases towards PG at 3eV

No real effect from the PG polarization. Other experiments decided to perform with grounded PG

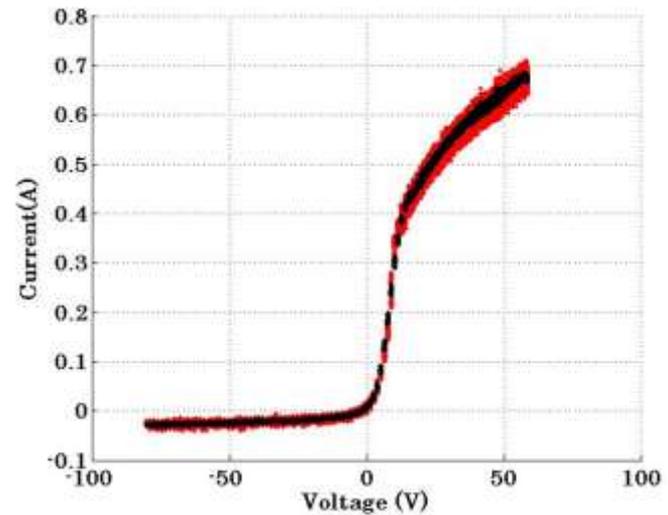
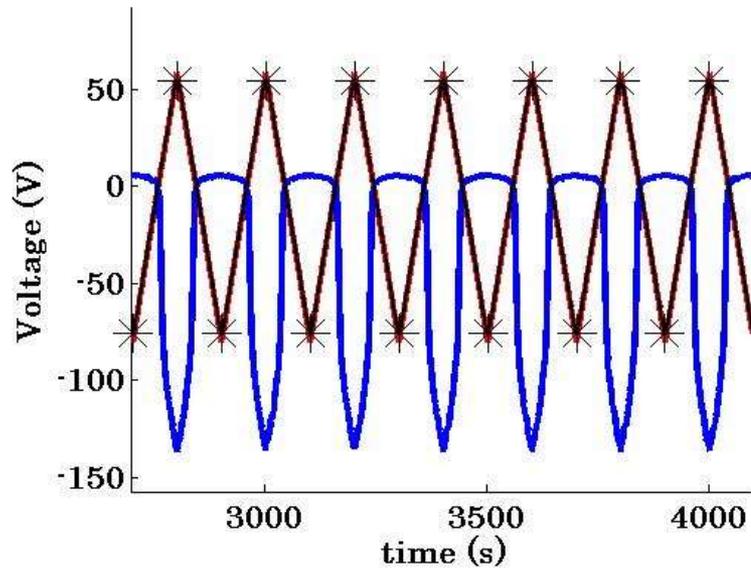
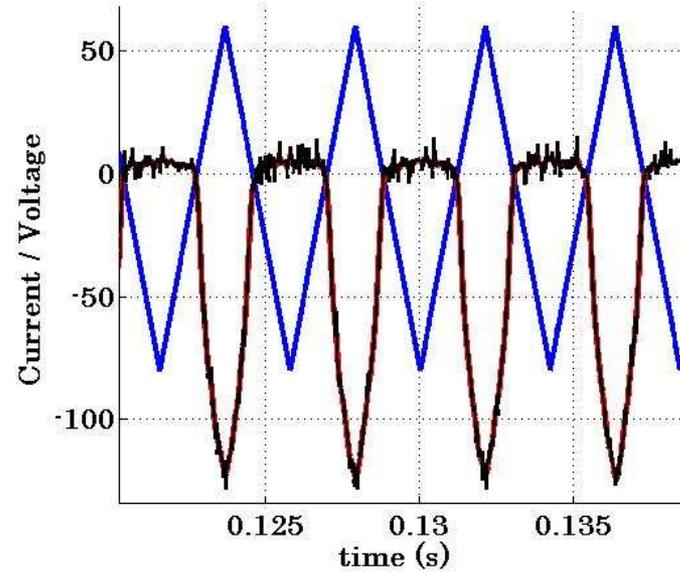
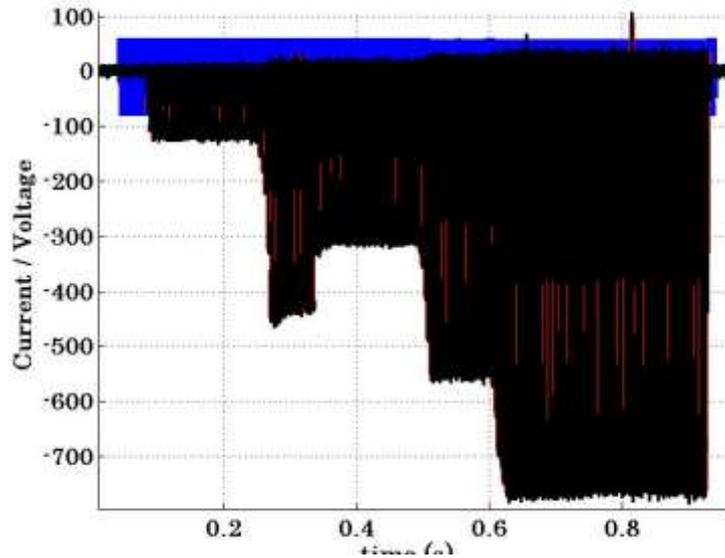


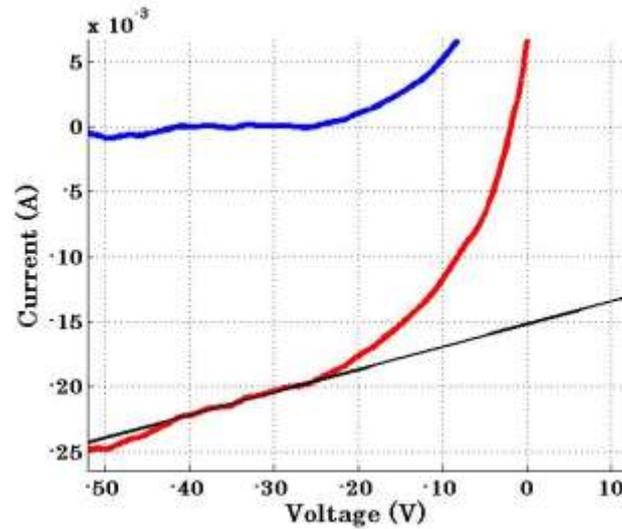
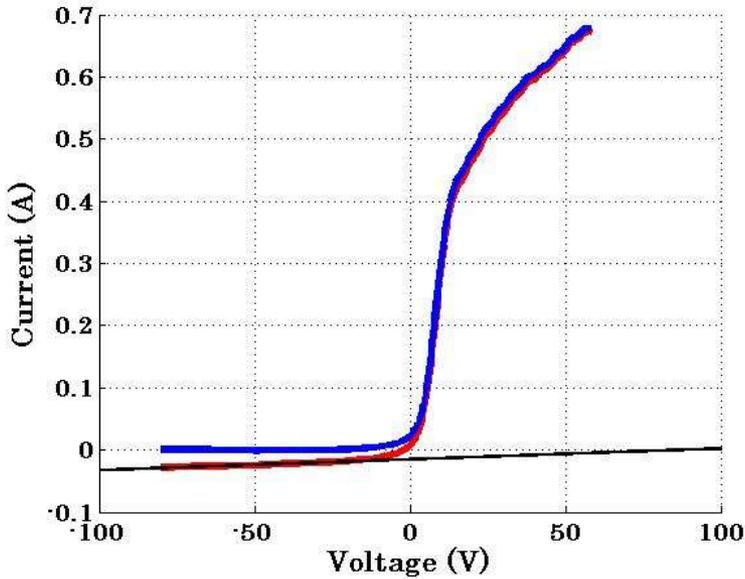
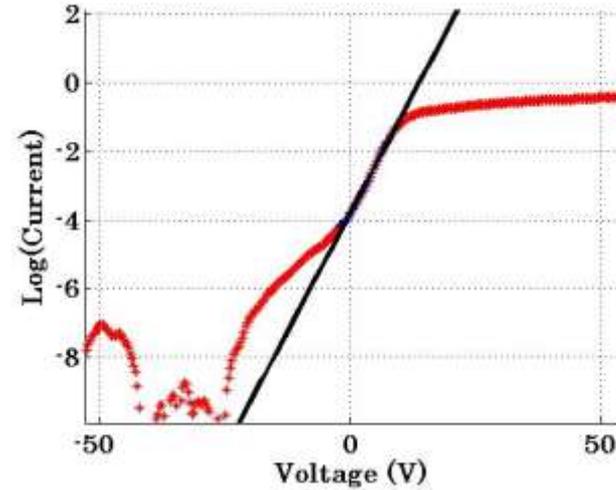
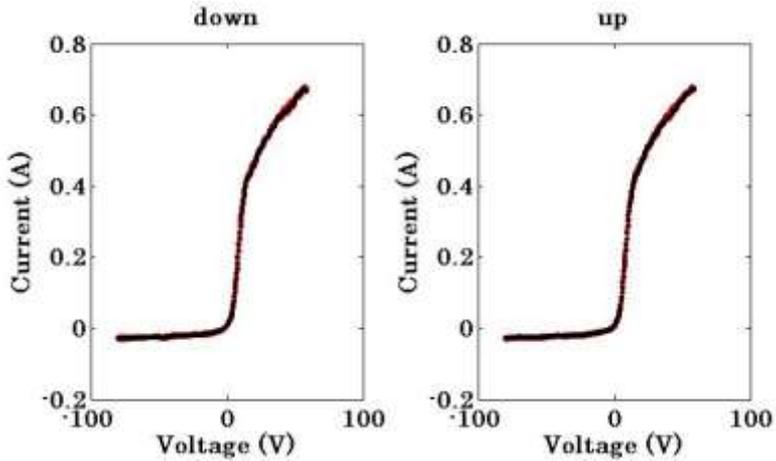
(60 cm from the Helicon driver, V bias -47V, Prf 3kW, p=0.3 Pa, B=100G, H).

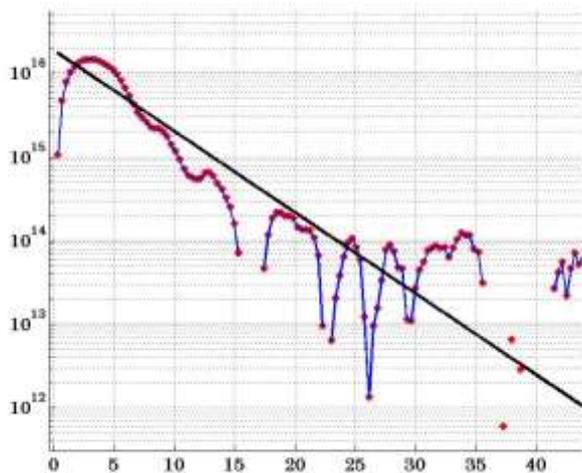
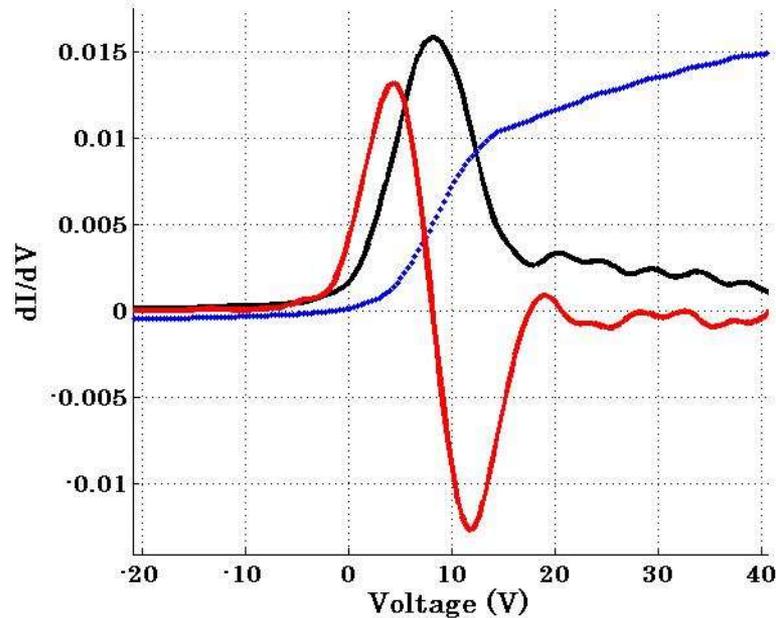


Different RF power  
Different gas (H<sub>2</sub> or Ar)

Above 16kV the  
breakdowns occur more  
frequently







$P_{rf} = 3\text{kW}$

Gas H,  $p=0.3\text{Pa}$

$B_{\text{field}} \sim 100\text{G}$  (280V set on the born)

PG grounded

V bias top and bottom plates = -55V

(350V set on the born)

probe ramp -80V : +60V

serial resistor 15 Ohm

Plasma\_potential = 8.8835 V

Float\_potential = -1.9139 V

Ion\_saturation\_current = -0.0155 A

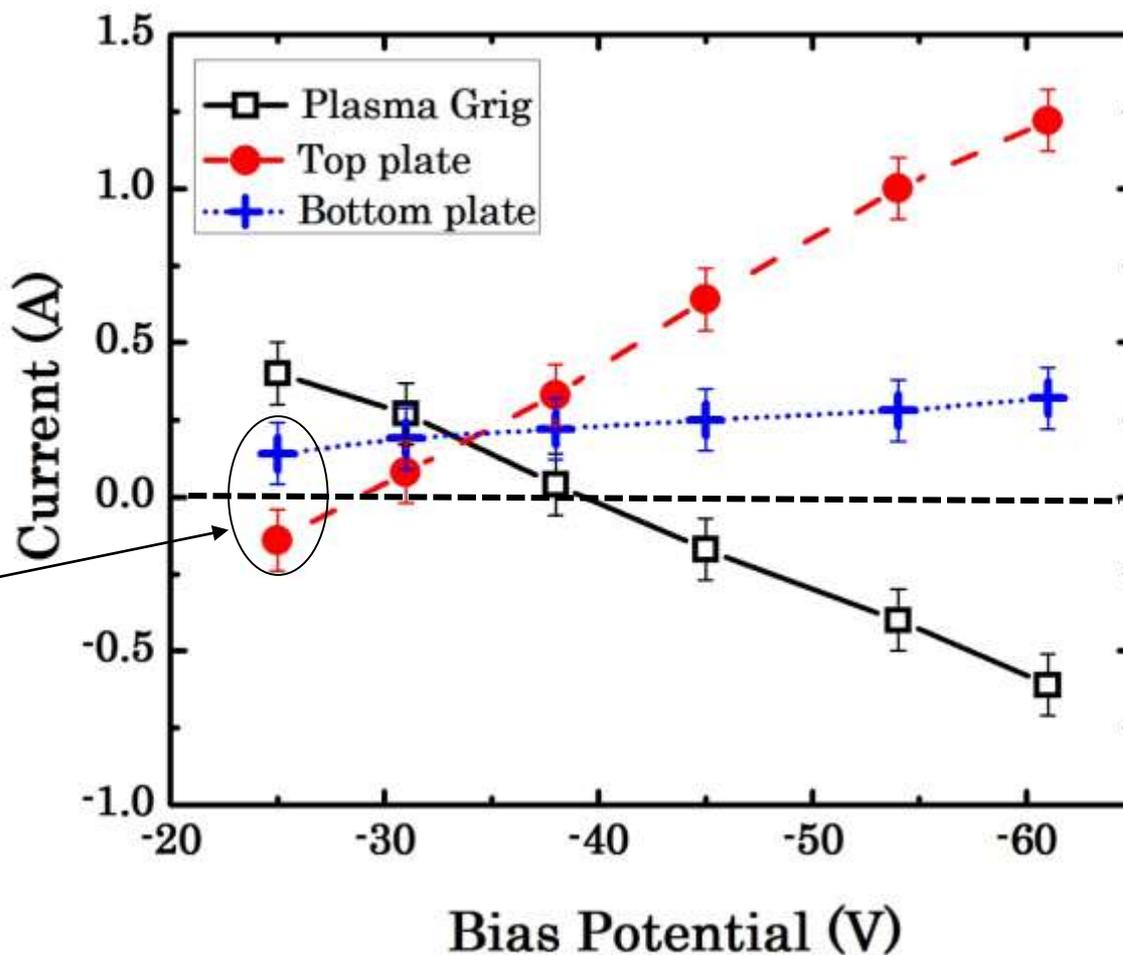
dens\_int =  $2.9865 \cdot 10^{17} \text{ m}^{-3}$

$T_{e\_int} = 3.9158 \text{ eV}$

dens\_EEDF =  $1.8521 \cdot 10^{17} \text{ m}^{-3}$

$T_{e\_EEDF} = 4.0506 \text{ eV}$

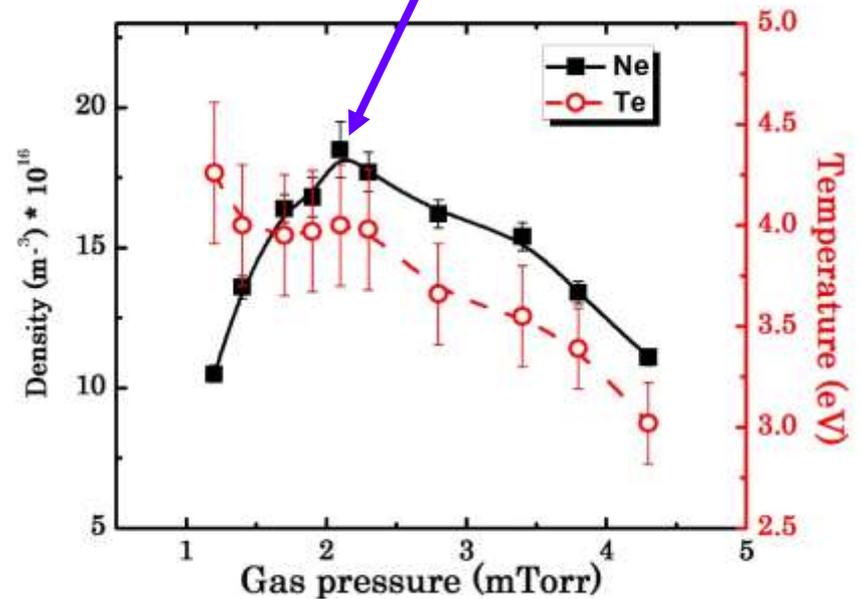
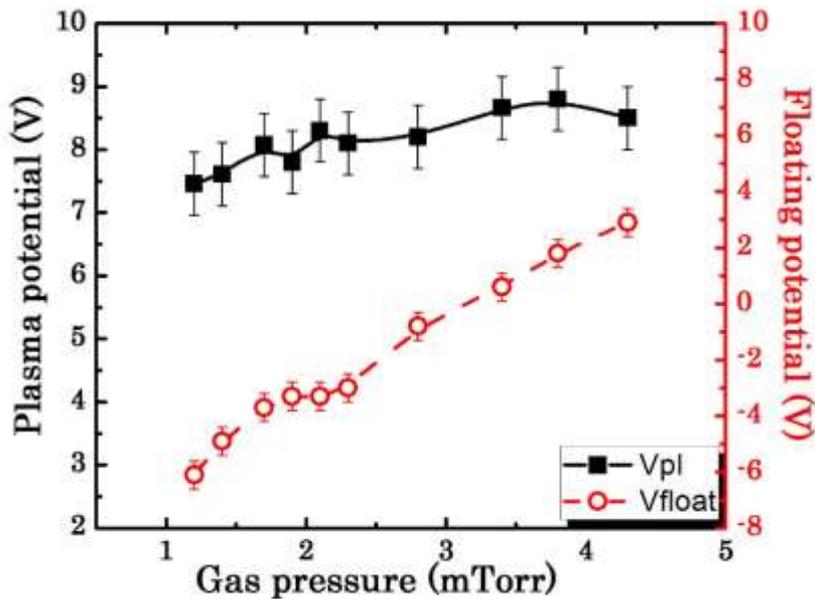
Current measured separately on the Plasma Grid, top and bottom bias plates  
**Variation of the bias potential.** Plasma Grid grounded



equal

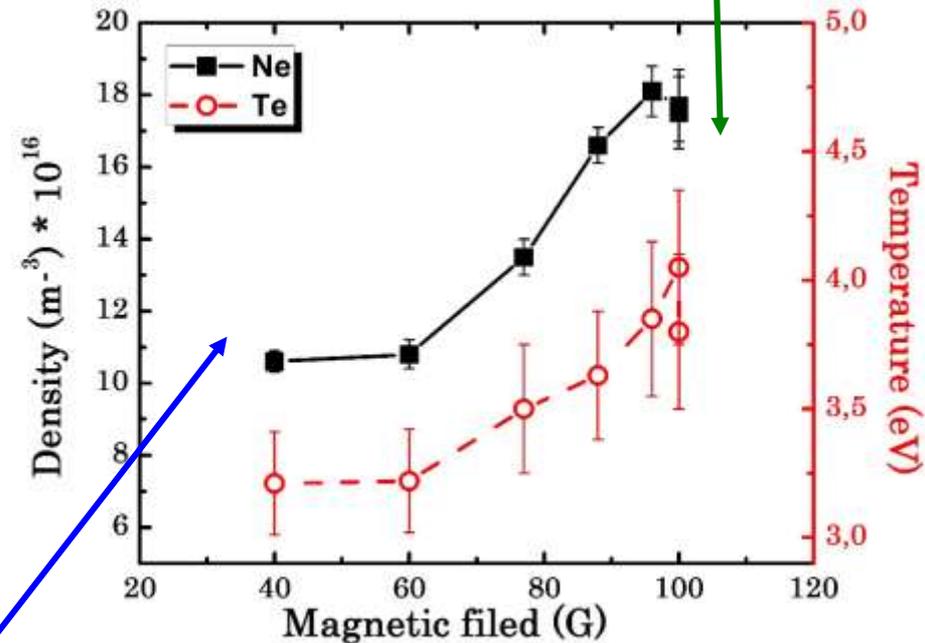
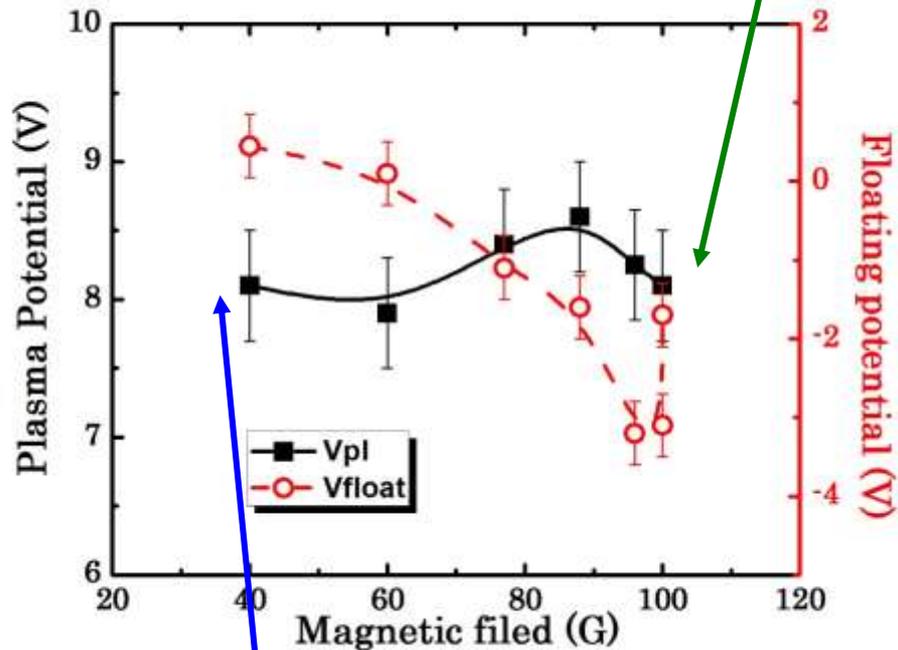
## Effect of the pressure

**Optimal pressure = 2.1mTorr (0.3Pa)**  
Highest plasma density



# Effect of magnetic field

Saturation of the frame coil  
(can not increase the Magnetic field higher that 100G)

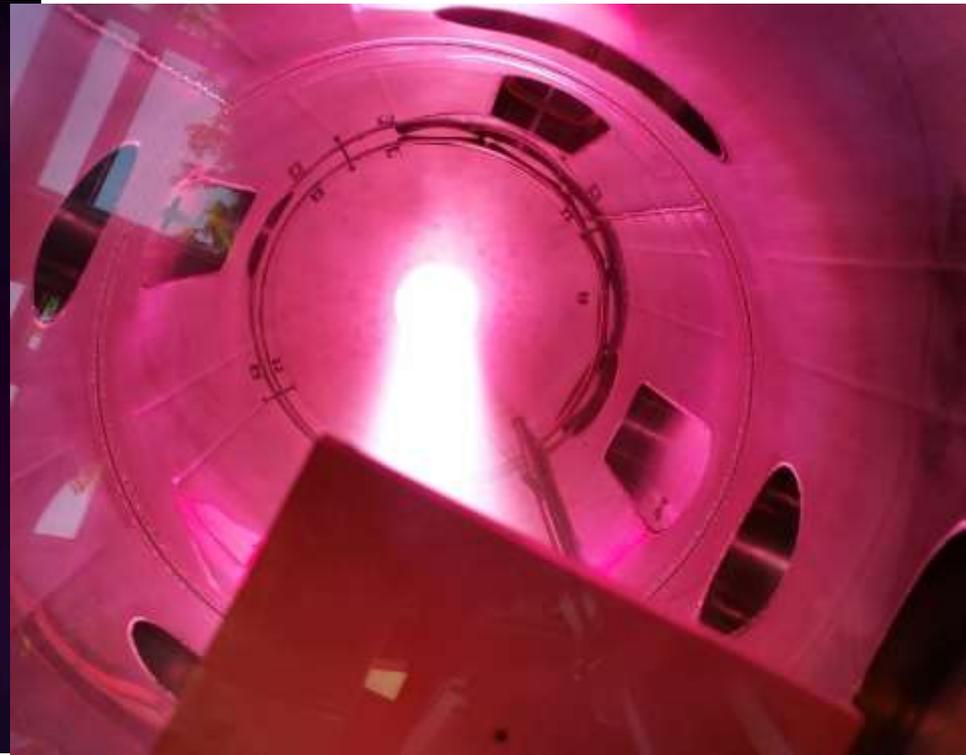
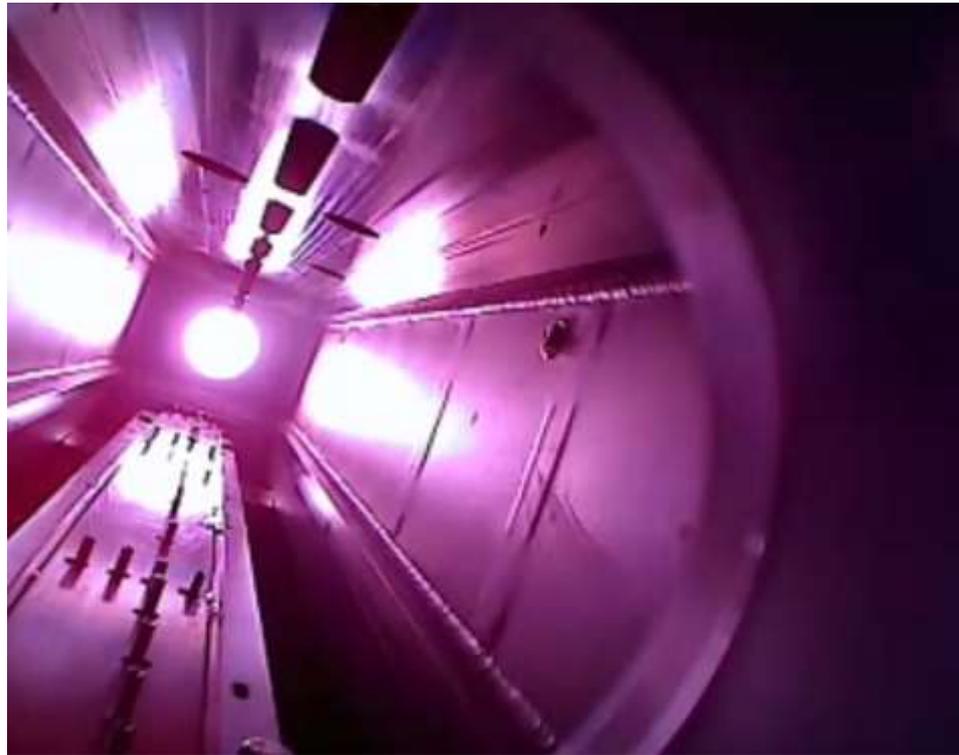


Magnetic field is not high enough  
to support the propagation of the helicon wave

## Comparison between ICP and Helicon

*ICP RF plasma on Cybele*  
 $P_{RF}=25\text{ kW}$ , no magnetic field

*Helicon plasma on the RAID testbed*  
 (EPFL)  $P_{RF}= 3\text{ to }5\text{ kW}$



**Plasma from ICP driver does not diffuse far in the Cybele source volume**

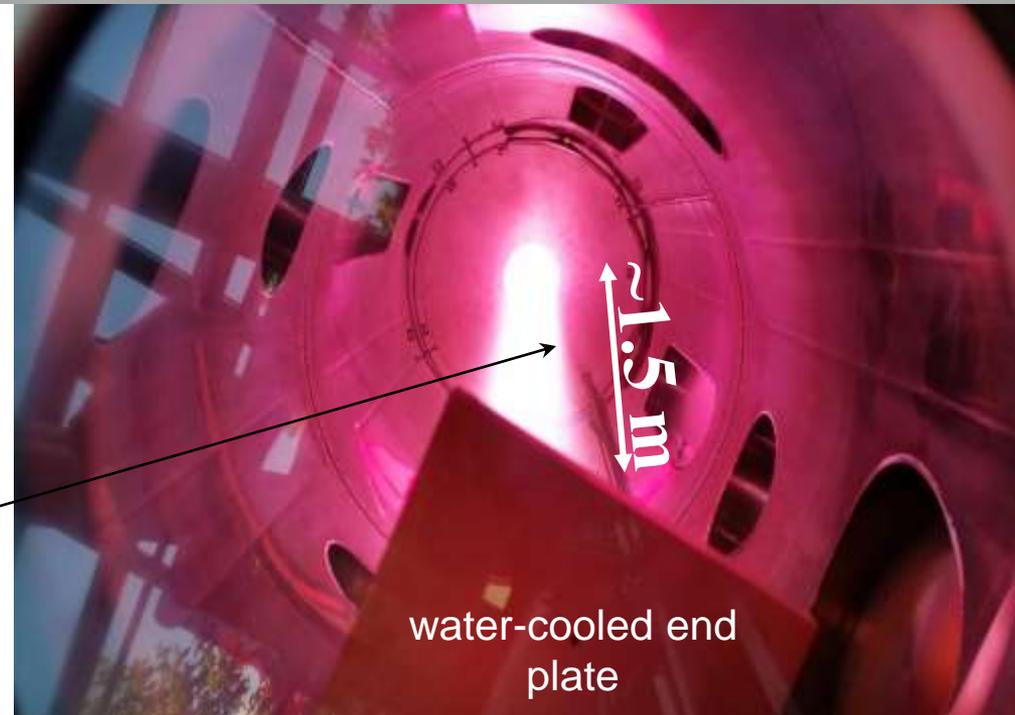
**Helicon plasma driver is essential for the magnetized plasma column of Cybele**

## Development of a 10 kW Helicon antenna (Bird-cage type) at RAID testbed (EPFL) to provide a dense magnetized plasma column

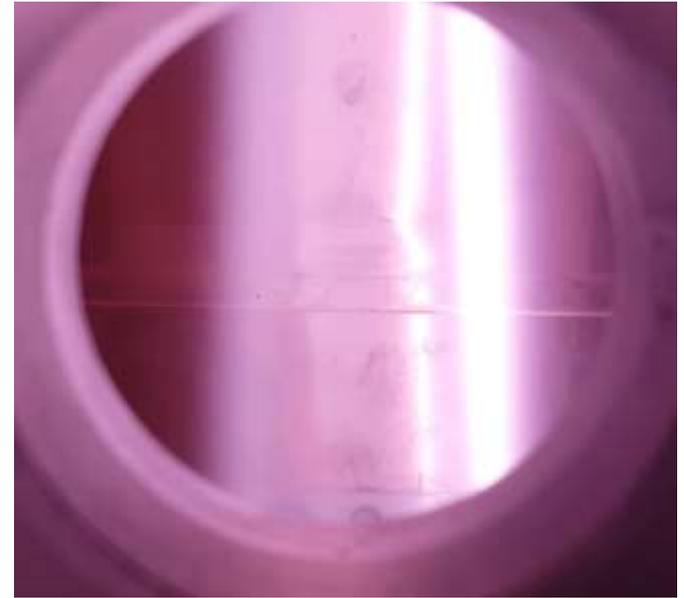
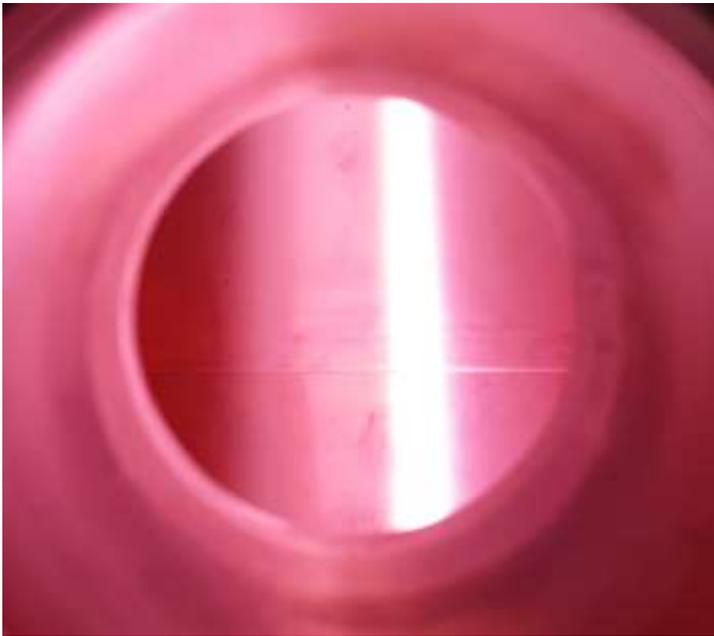
Helicon Bird-cage antenna meets the specifications:

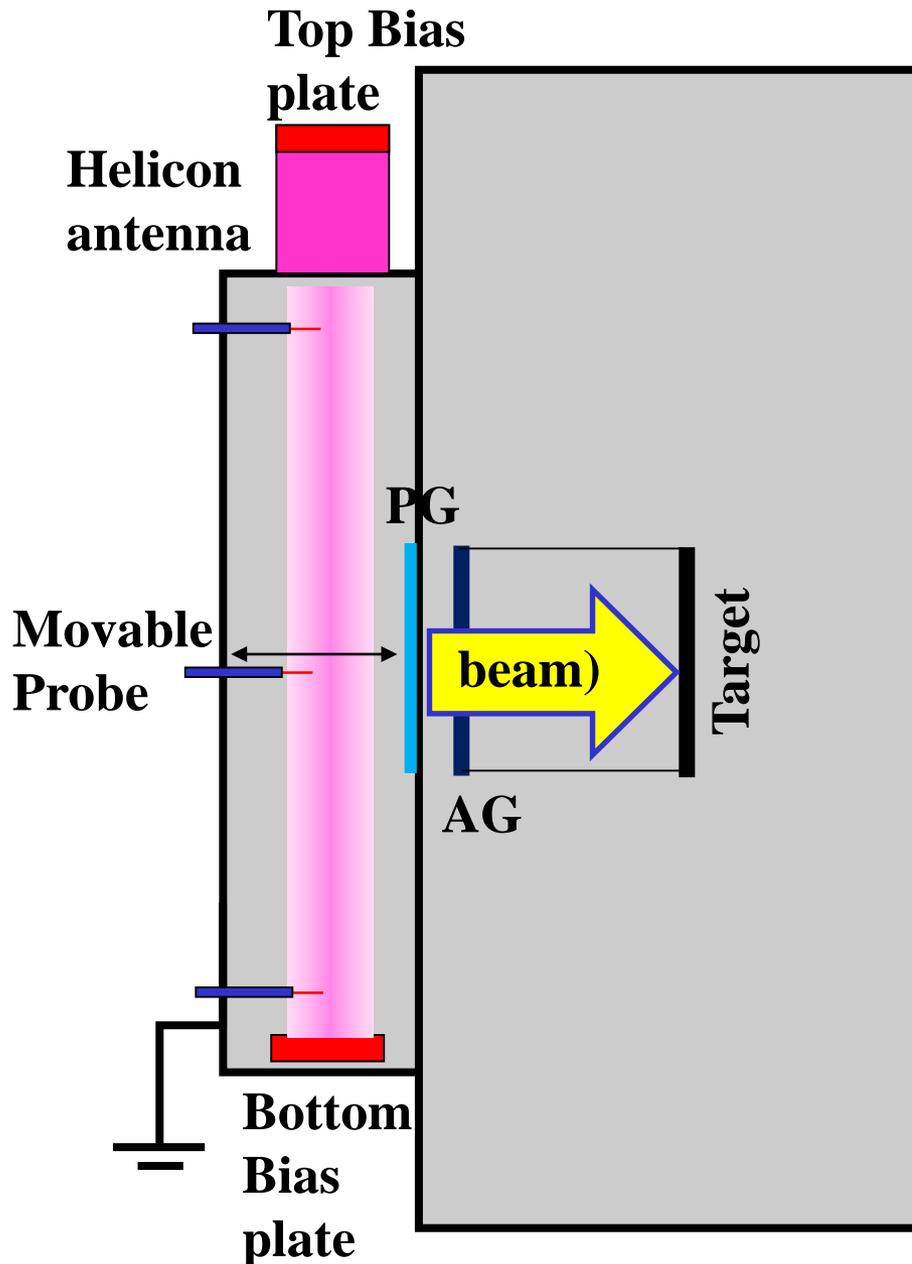
A 3 kW, 0.3 Pa,  $B = 12$  mT,  $H_2$  plasma jet

- Low B-field ( $\sim 10$  mT)
- Low operating pressure ( $\sim 0.2$  Pa)
- Stable plasma discharges in  $H_2$  and  $D_2$  up to 10 kW plasma (achieved)
- Nearly constant section
  - ⇔ Uniform plasma distribution along  $B_{\parallel}$



**Plasma instability??  
Conditioning??**

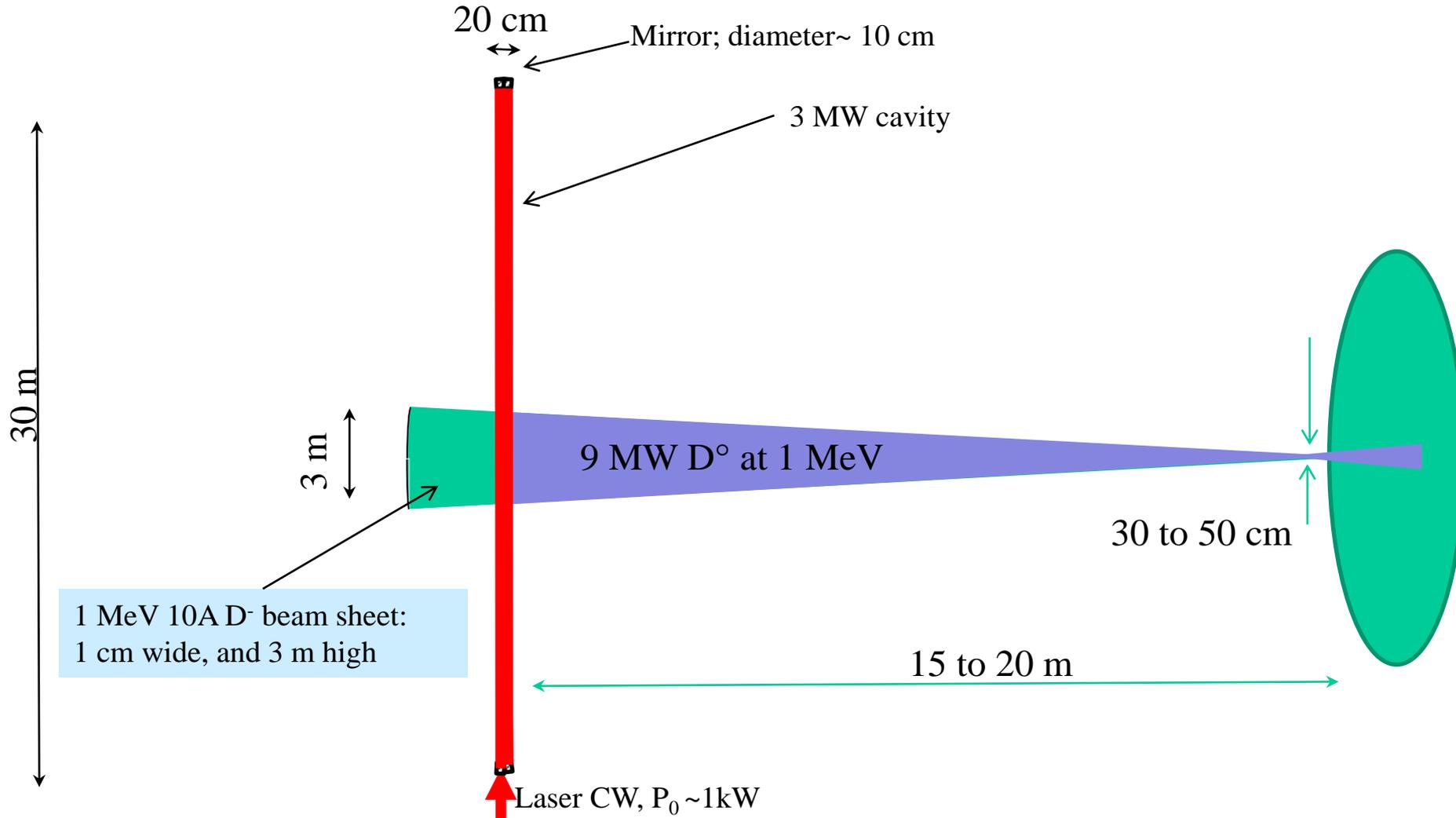




Cybele with **negatively biased Top and bottom plates** and **Plasma grid** (grounded, **floating** or **positively polarized +5V**)

Magnetic field – 100 G  
 RF power – 3kW  
 Gas pressure - ~ 0.3 Pa  
**Bias plates : -25V : -90V**  
**Plasma Grid : grounded, floating , +5V**  
 Probe sweep – [-80V : 60V]  
 Sweep frequency – 10 Hz

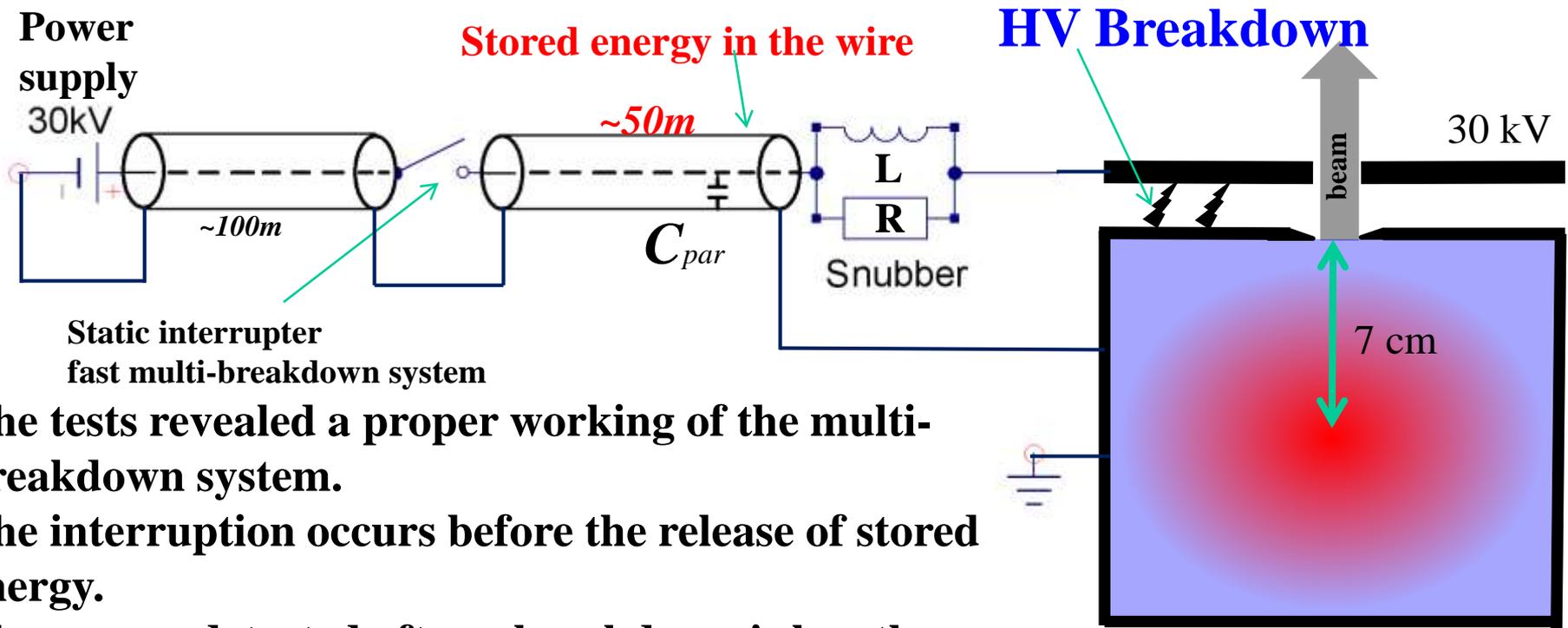
# Side view of one beam-sheet with a single 3 MW cavity



# Electrical setup installation of the 30 kV pre-accelerator

With High Voltage breakdown – risk of damaging PG, arcing etc.

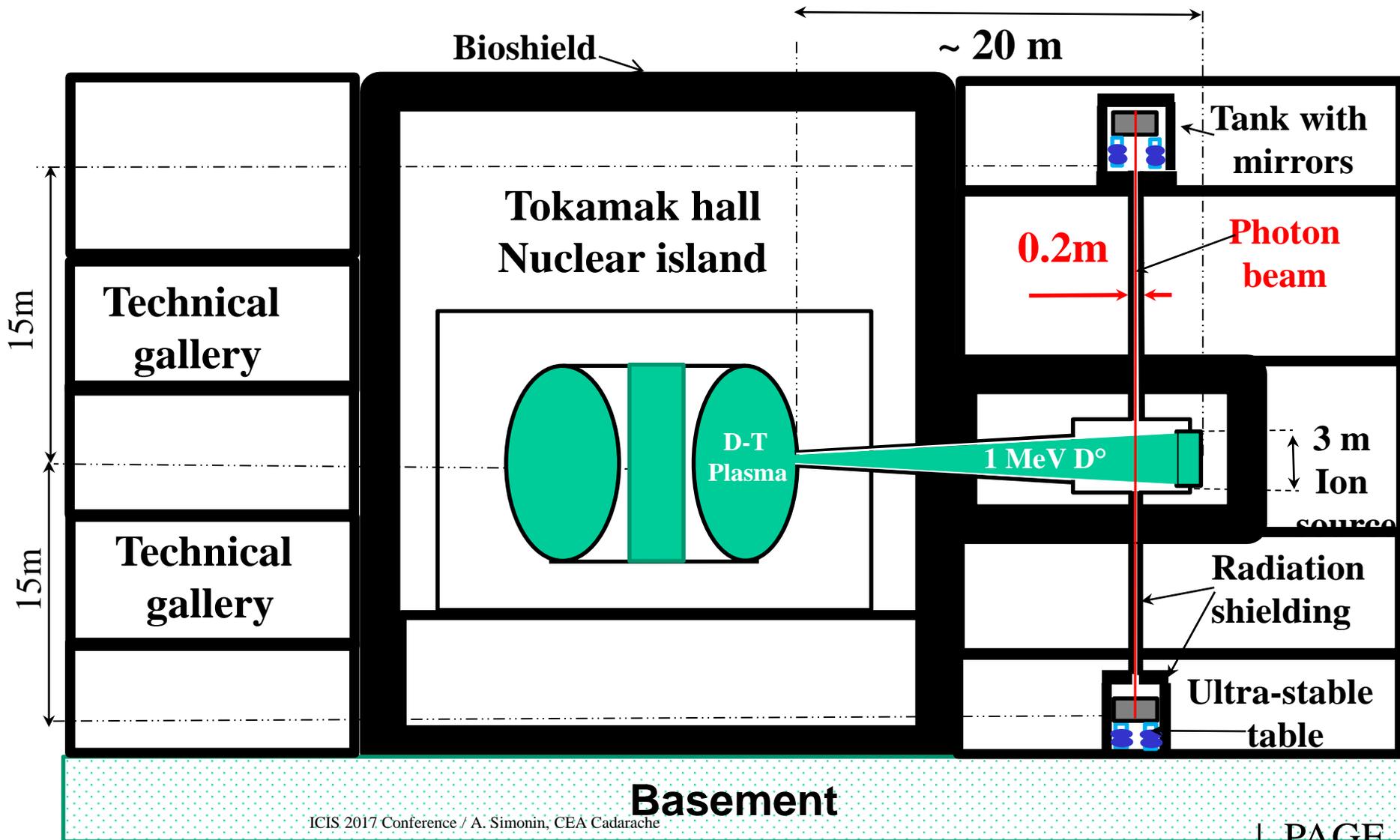
- ⇒ Interruption of the current in the  $\mu\text{s}$  range (fast switch multi-breakdown system)
- ⇒ Removing the stored energy in the HV wires by a snubber (to avoid grid damages)
- ⇒ The reset of the Static interrupter in the 10 ms range
- ⇒ HV holding and beam conditioning involves several tens of HV breakdowns per second



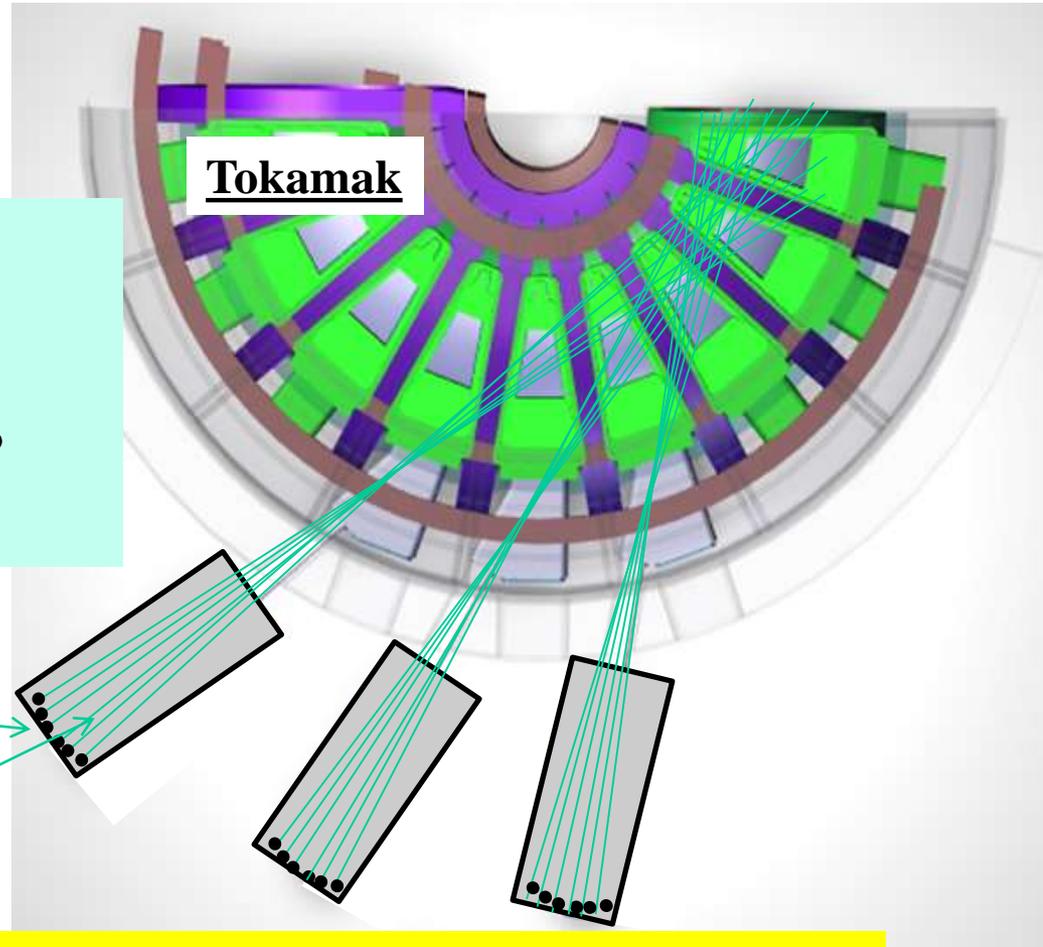
The tests revealed a proper working of the multi-breakdown system.

The interruption occurs before the release of stored energy.

The energy detected after a breakdown is less than 5mJ.



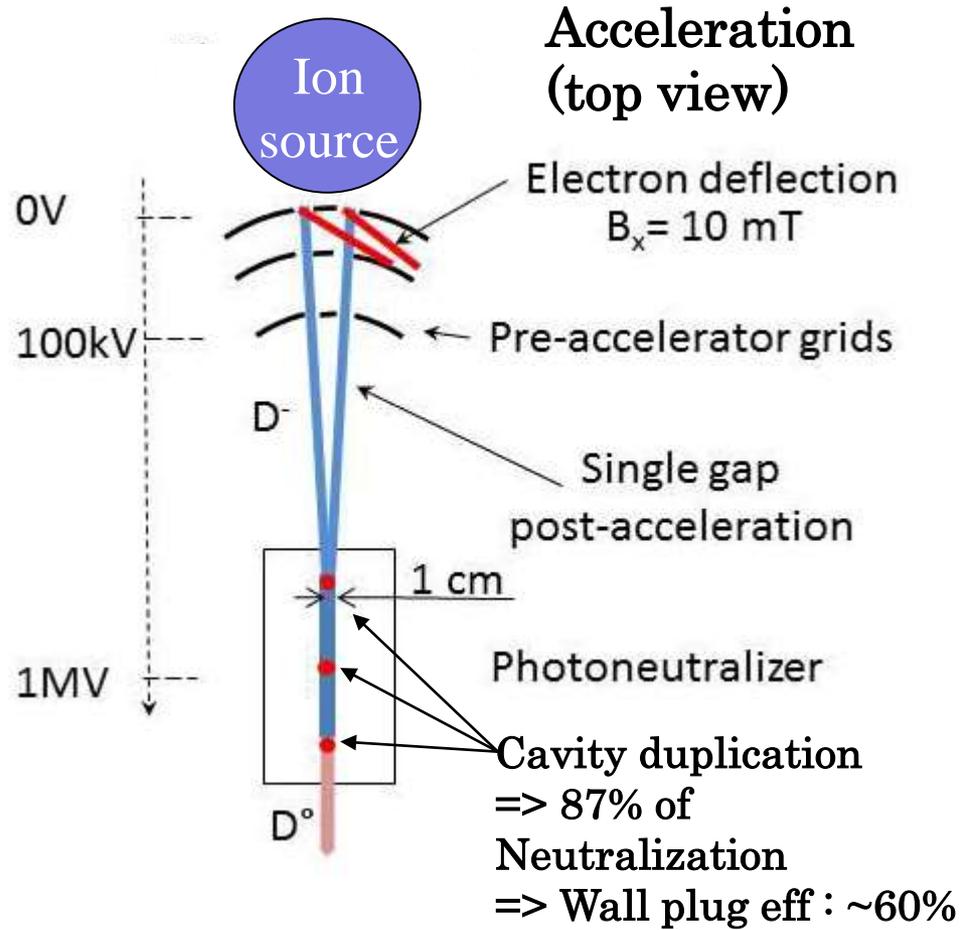
**Modular concept**  
 ⇒ Six beamlines in // per tank  
 ⇒ 50 MW D° per tank  
 ⇒ Three tanks in //: ~ 150 MW D°  
 ⇒ Overall efficiency: ~70%



NB vacuum Tank

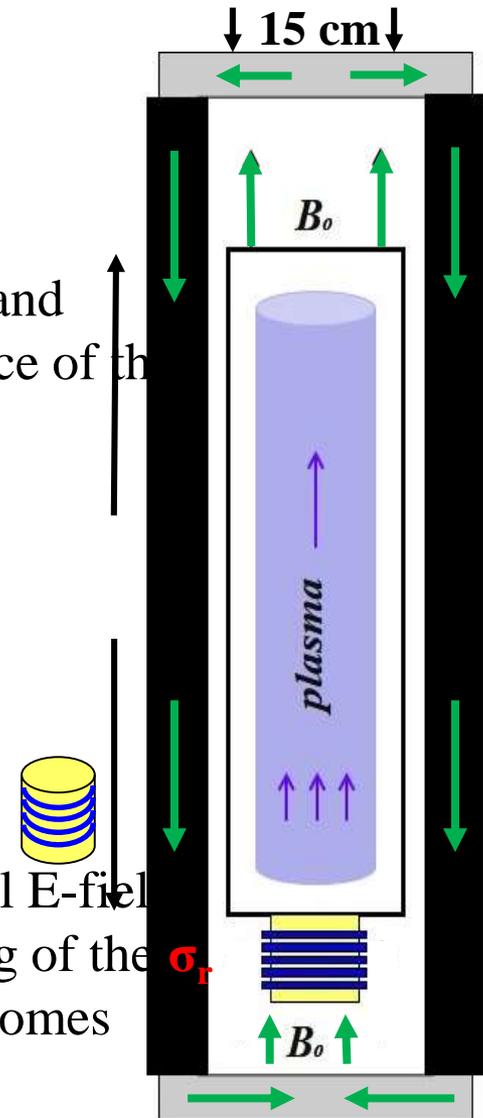
Six beam sheet in // per tank

**Photo-neutralization allows to achieve powerful neutral beam with high efficiency**

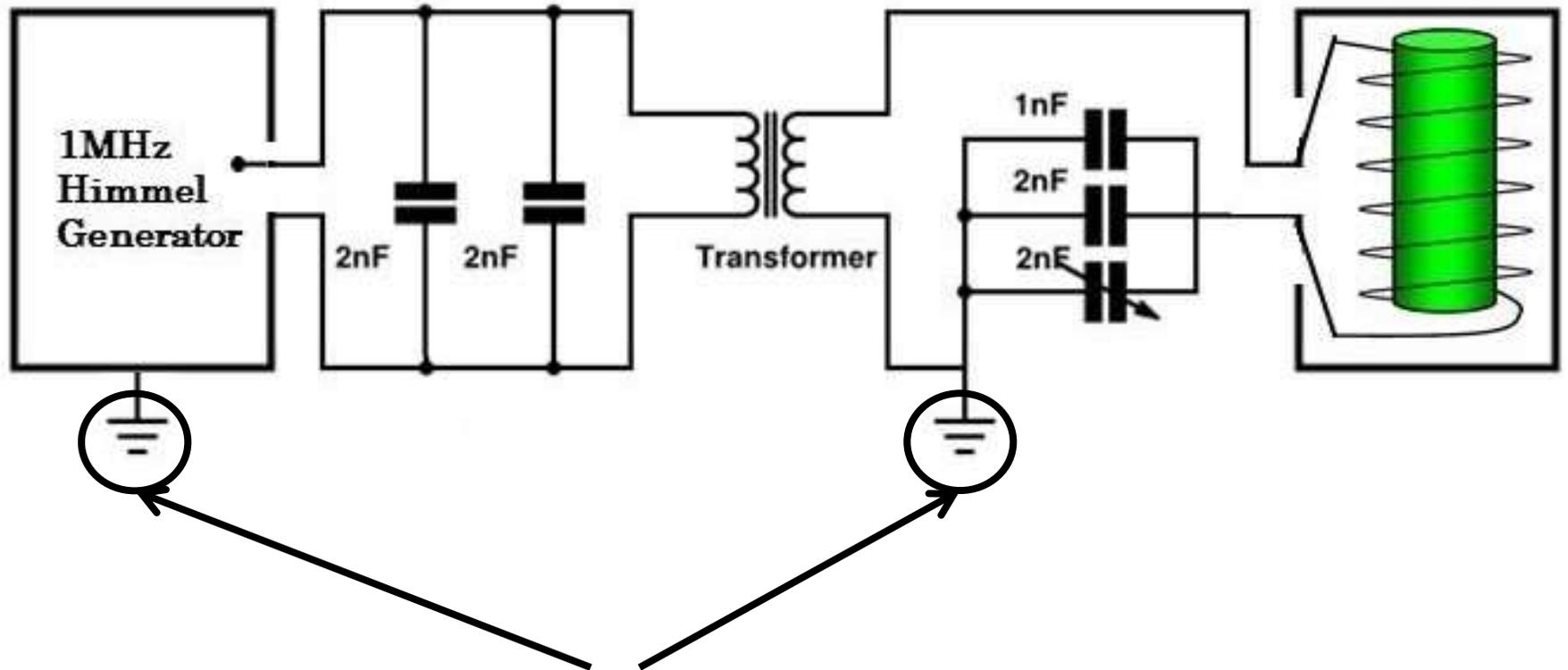


Vertical B-field increases vertical diffusion => radial diffusion and conductivity decreases => increasing of the skin-depth => reduce of the RF-induced current

The electrons in ICP driver are accelerated by the RF azimuthal E-field ( $E_\phi$ ) experience a radial Lorentz force ( $\mathbf{F}_r = \mathbf{v}_\phi \times \mathbf{B}_z$ ) => reducing of the => decreasing  $I_{\text{plasma}}$ . => For B-fields larger than 2.5 mT, it becomes impossible to couple the RF active power to the plasma.



## RF electrical set up



*Ground Decoupling between the RF generator and the antenna circuit*

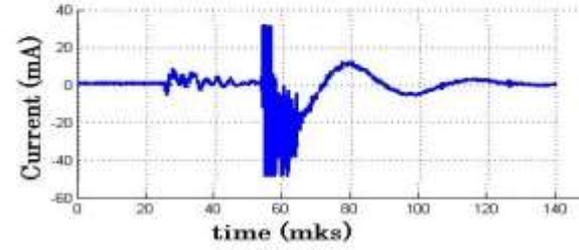
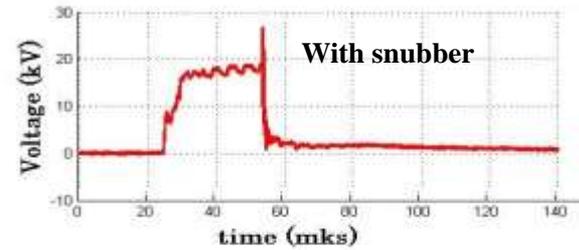
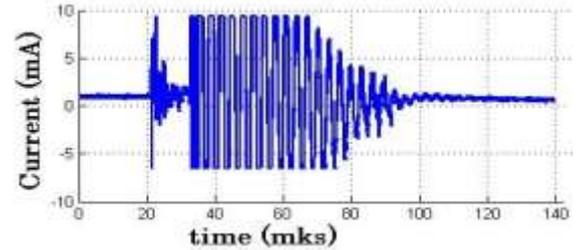
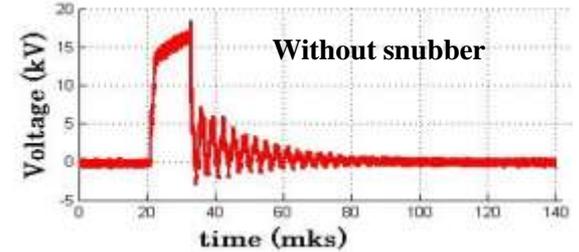
**Static interrupter  
fast multi-breakdown system**



2m

**Snubber**

70 cm



**The tests revealed a proper working of the multi-breakdown system.  
The interruption occurs before the release of stored energy.  
The energy detected after a breakdown is less than 5mJ.**

## Matching impedance

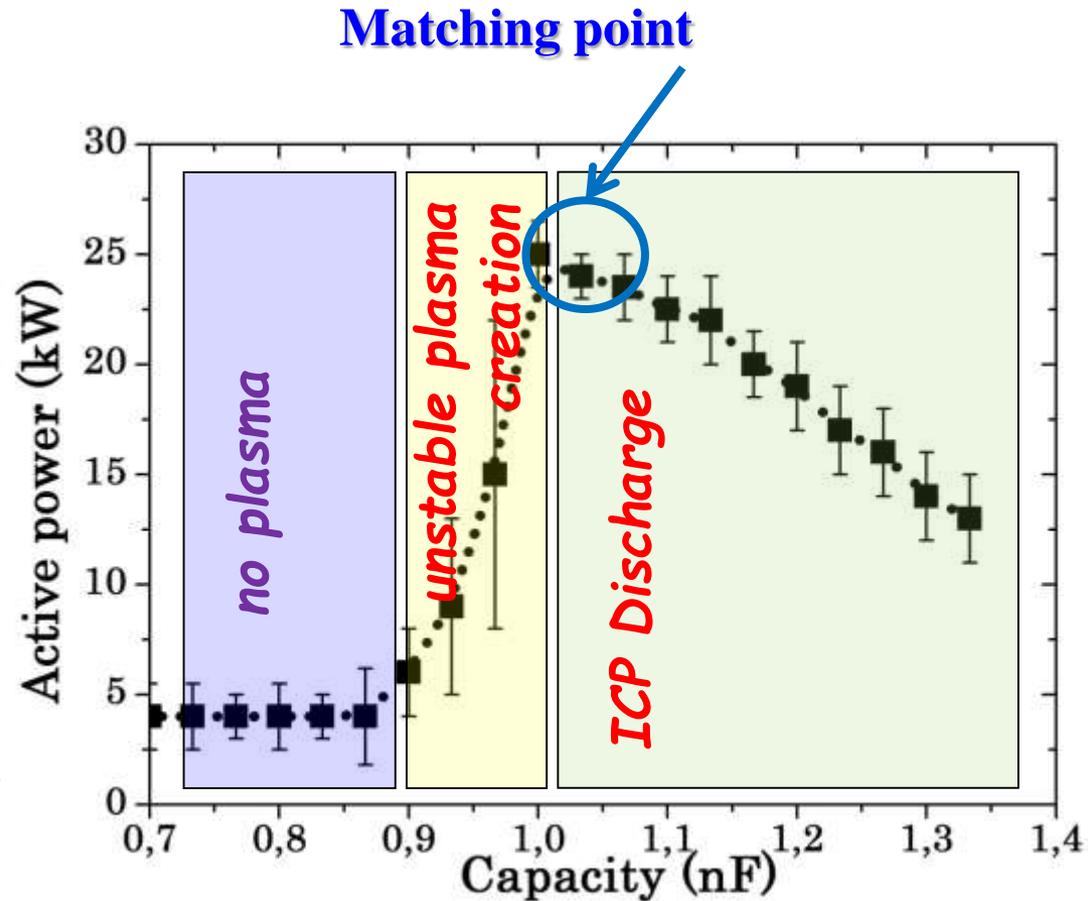
Gas – Hydrogen

Pressure – 0.3 Pa (ITER condition)

Set point of the generator power : 30 kW

Measured Frequency : ~0.94 MHz

Active power coupled to the plasma 23-26 kW at the matching (~ 0.75-0.85 of total power)





## Photo-neutralization seems ideal

- No gas injection => Strong reduction of D<sup>-</sup> losses
- Clean : No pollutant
- Potential High neutralization rate ( $\eta > 90\%$ )

## But

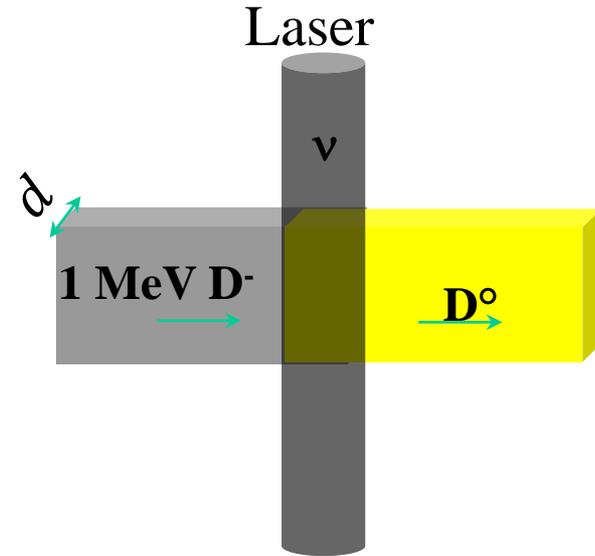
- **Low photo-detachment cross-section**

$$\sigma \sim 3.6 \text{ to } 4.5 \cdot 10^{-21} \text{ m}^2 \text{ for } \lambda = 1064 \text{ nm}$$

**Photo-neutralization requires high photon power !!**

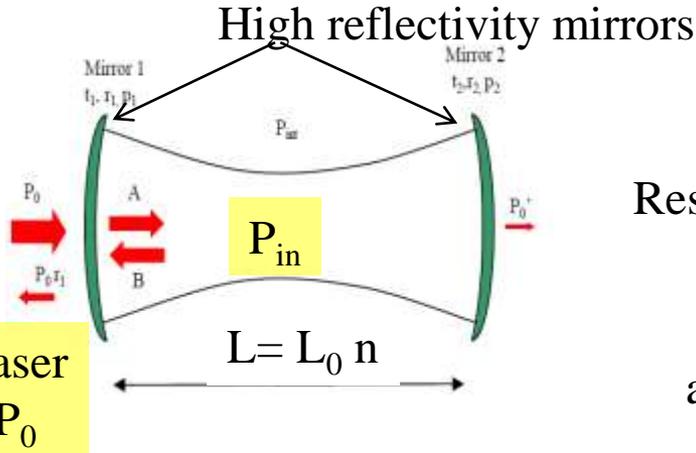
# Evaluation of the photon power

- *1 MeV D<sup>-</sup> blade-like beam*
- *D<sup>-</sup> beam width:  $d \sim 1\text{cm}$*
- *50 % photo-detachment rate*



D<sup>-</sup> Ion velocity,  
 $|v| \sim 10^7 \text{ m/s at } 1\text{MeV}$

$$P_{\text{photon}} = hc \cdot \frac{|v| d}{\sigma \lambda} \quad \left. \vphantom{P_{\text{photon}}} \right\} P_{\text{photon}} \sim 3 \text{ MW}$$



Resonance  $\Leftrightarrow 2 L = q \lambda \Leftrightarrow$  Constructive interferences

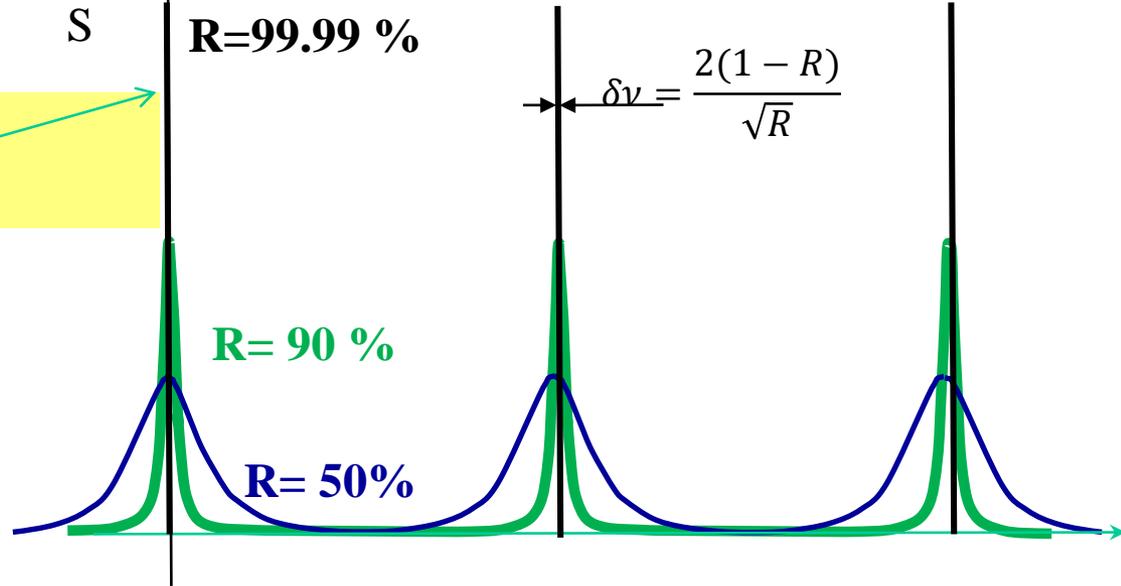
Cavity amplification  
S

**R=99.99 %**

**R= 90 %**

**R= 50%**

$$\delta\nu = \frac{2(1 - R)}{\sqrt{R}}$$



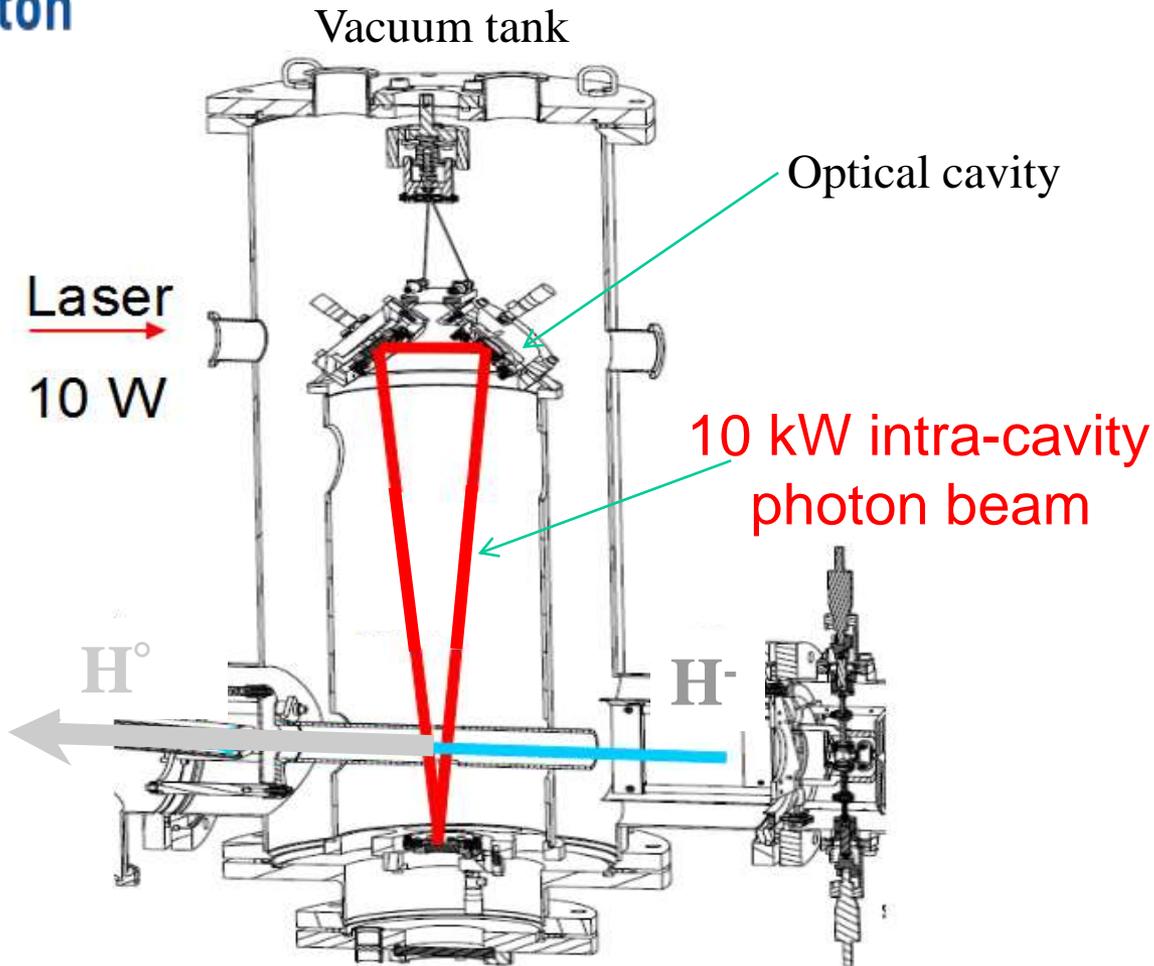
Photon power stored within the cavity:  
 $P_{in} = P_0 \times S$

$P_{in} \nearrow \Rightarrow \delta\nu \searrow$

High cavity sensitivity to variations of the optical length: vibrations, etc.

# Photo-neutralization experiment in cavity

~1 m



➤ H<sup>-</sup> beam: 1 keV and 1 mm diameter

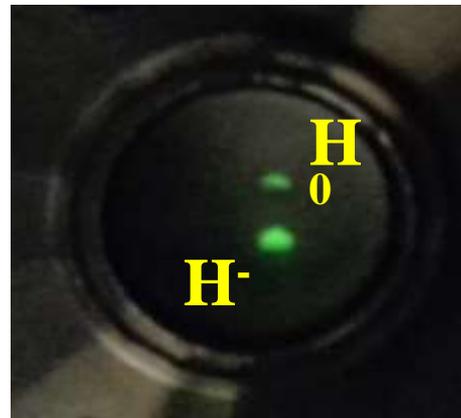
# Photo-neutralization experiment in cavity

## Preliminary results

Photon  
power  
0 kW

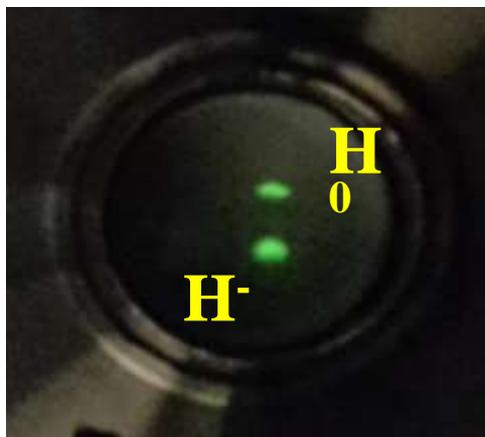


Photon  
power  
13 kW



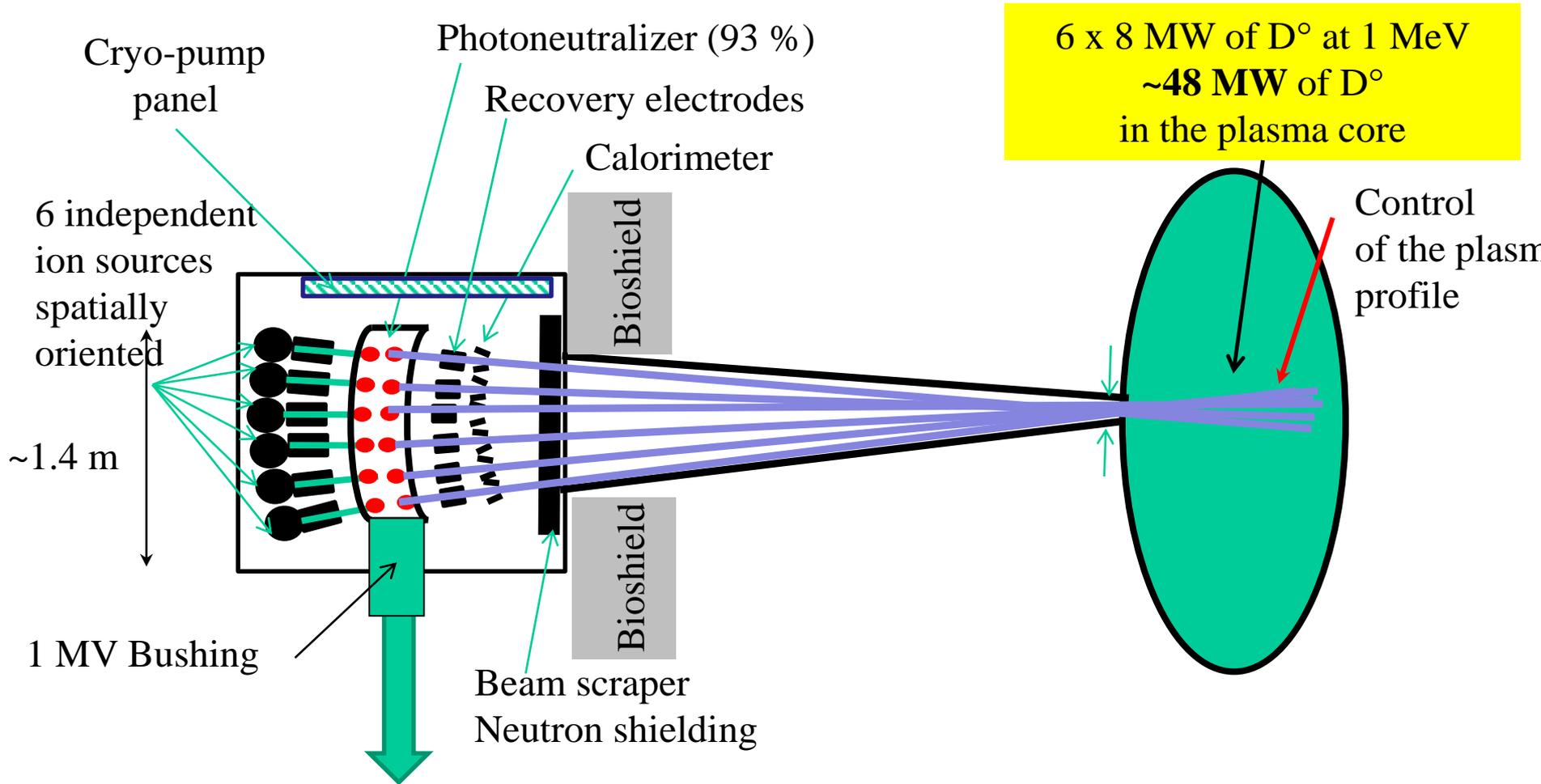
### Observation of $H^-$ and $H^0$ on micro-channel detectors

Photon  
power  
23 kW



⇔ **50 % photo-detachment achieved in CW regime**

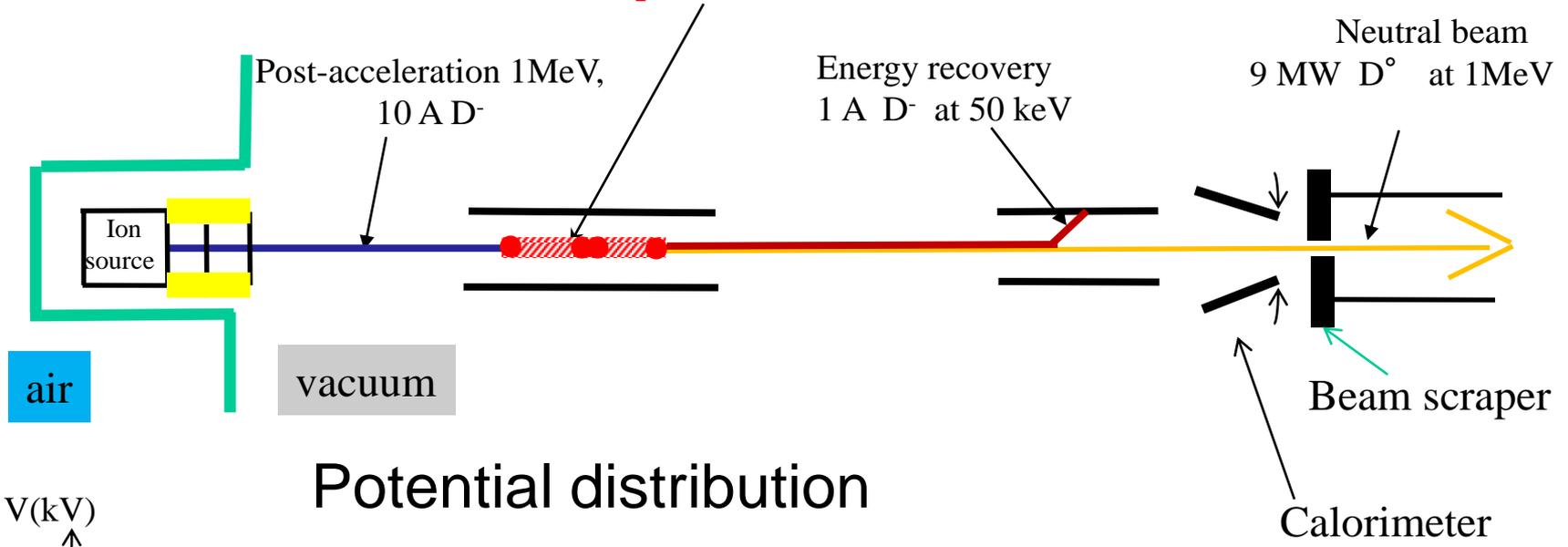
**Publication:** « Saturation of the photoneutralization of a  $H^-$  beam in continuous operation »; D. Bresteau, C. Blondel, C. Drag; Rev. Sci. Instrum.; 2017; in press.



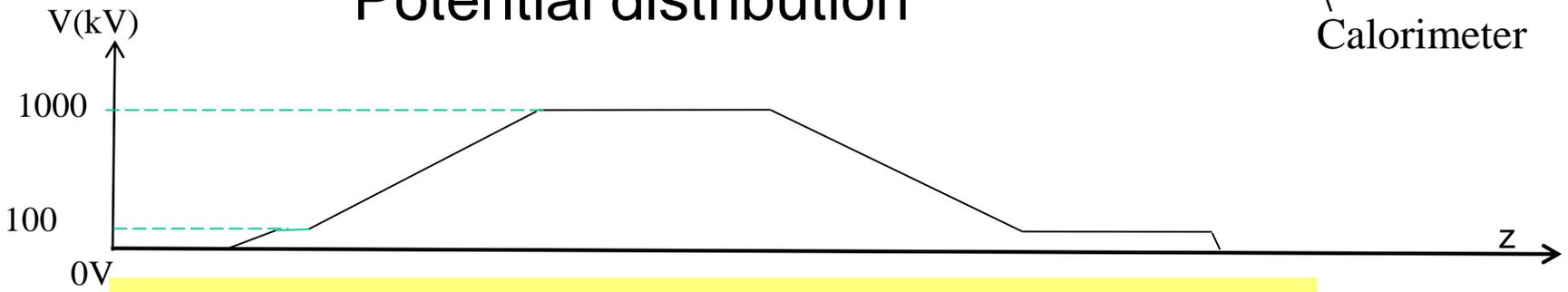
# One SIPHORE beam-sheet principle

(Top view)

Photon beam  $\Leftrightarrow$  93% photodetachment



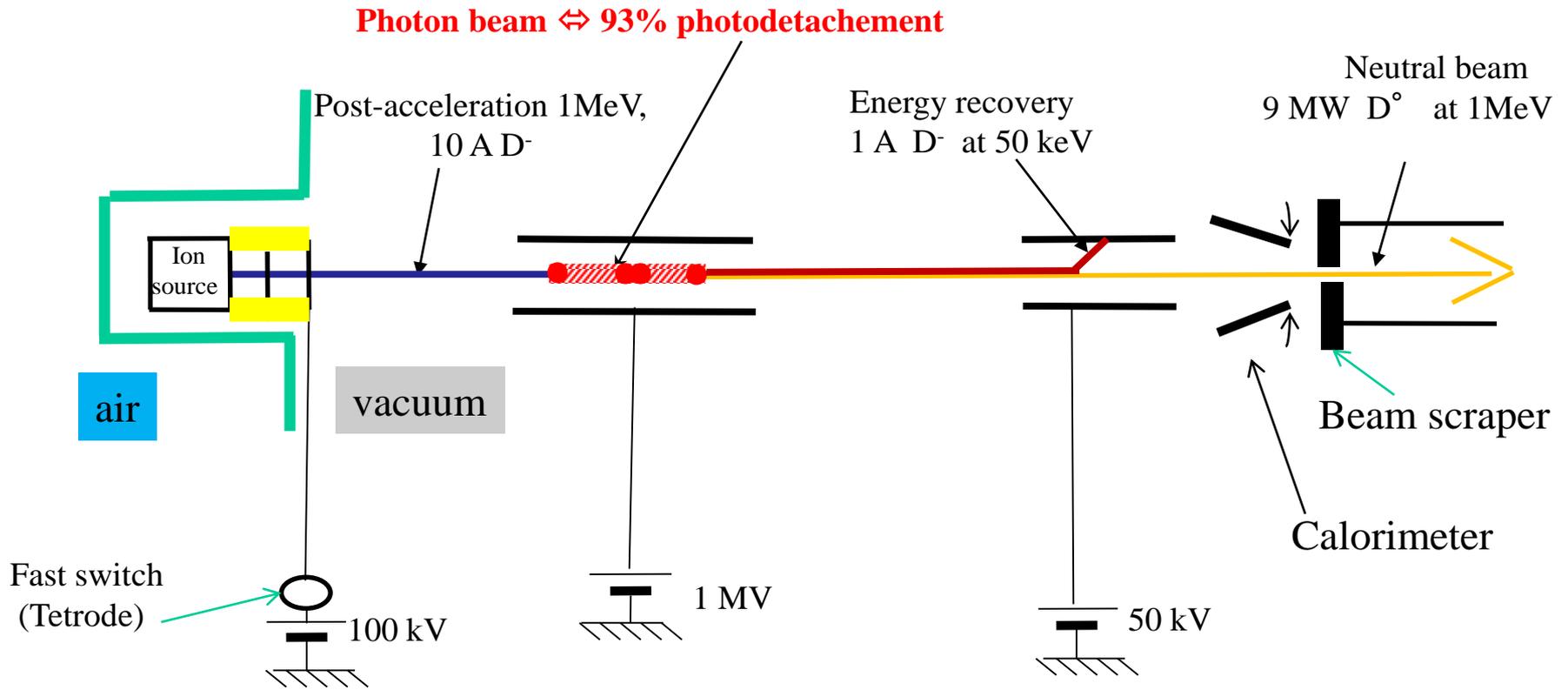
## Potential distribution



The ion source and pre-acc. are referenced to the ground potential

# One SIPHORE beam-sheet principle

(Top view)



## Ion source grounded $\Rightarrow$ Huge simplification of the electrical set-up

- Fast switch in the pre-accelerator allows to switch on /off the 10 A  $D^-$  beam in **the  $\mu s$  range**
- It is conventional technology of present NBI systems (JET, etc.)
  - **Temporal modulation of the  $D^0$  beam**