

# Modelling of beam transport and interactions with beamline components in the CFETR neutral beam test facility

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- **CFETR neutral beam test facility**
- **Conceptual design of beamline**
  - ▶ Beam transport
  - ▶ Beam deposition
  - ▶ Thermo-mechanical analysis
- **Summary**



# Chinese Magnetic Fusion Development



## Roadmap for Chinese MCF Development





# Chinese Fusion Engineering Test Reactor



- ▶ Demonstration of a full cycle of fusion energy with a fusion power of 200-1000 MW
- ▶ Demonstration of long pulse or steady-state operation with duty cycle of 0.3–0.5
- ▶ Demonstration of a full fuel cycle of Tritium aiming at a tritium breeding ratio (TBR) >1.0

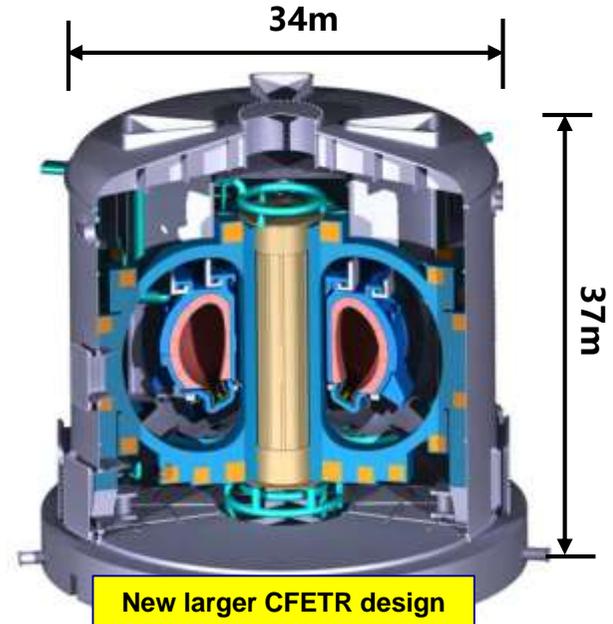
	CFETR		ITER	
Major Radius(m)	7.0		6.2	
Minor Radius (m)	2.2		2.0	
Elongation	2.0		1.86/2.0	
Fusion Power (MW)	200	1000	500	350
Burning Time (sec)	Steady-state	Steady-state	≤400	≤3600
Duty Circle	0.3-0.5	0.3-0.5	0.25	0.25
Q-value	3	17	10	5
Toroidal Field (T)	6	6.5	5.3	5.18
Plasma Current (MA)	12	13	15	9
Bootstrap Fraction	0.4	0.6	--	0.5
Additional Heating (MW)	69	58	50	59



## H&CD system for CFETR

- ▶ Phase I ( $P_{add}=69$  MW): **Off-axis NBI** + Top launch ECRH (170-230GHz)
- ▶ Phase II ( $P_{add}=58$  MW): **Off-axis NBI** + Top launch ECRH (170-230GHz) + LHCD (5-7.5GHz) at high field side
- ▶ NB energy and power is not determined for CFETR, but will be **similar to (or more demanding than) ITER-NBI requirement.**

**>0.8MeV, >30MW, >1h**  
**NB heating is required for CFETR**



ITER operation scenarios	NB	RF (ICRH+ECRH)
Inductive operation (500MW@400s)	34 MW (H)	16 MW
Steady state operation (350MW@3600s)	30 MW (D)	29 MW
Initial installed	33 WM (D)	20+20 MW

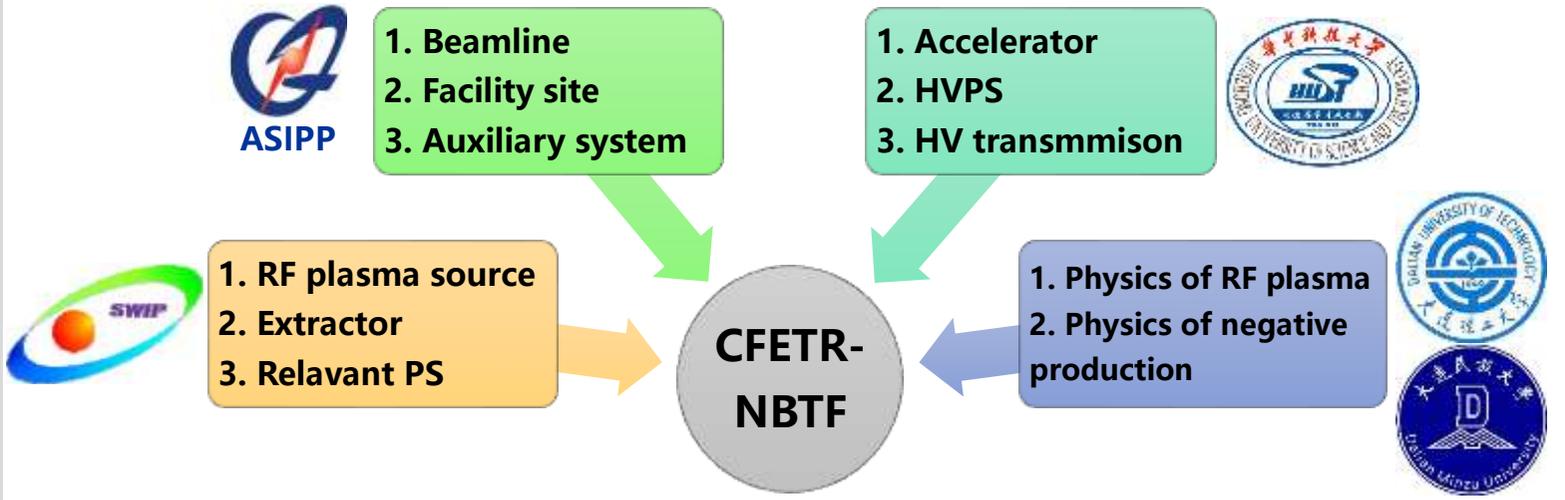


# Towards CFETR NBI



## CFETR Neutral Beam Test Facility (National Key R&D Program of China)

- ▶ **Object:**
  1. **Multi-RF-driver based Negative Ion Source (NIS) with single stage accelerator** for NI production & extraction, NI beam acceleration & steering
  2. **Demonstration of beamline system** for NI beam transmission & neutralization, residual neg.&pos. ions separation, heat transfer enhancement, cryopump
  3. **200kV HV P. S.**, HV transmission, HV holding technology
- ▶ **Goal:** H<sup>0</sup> beam, 1.6×0.32m<sup>2</sup>, 200keV, >2MW, 3600s





# Basic designs of NBTF

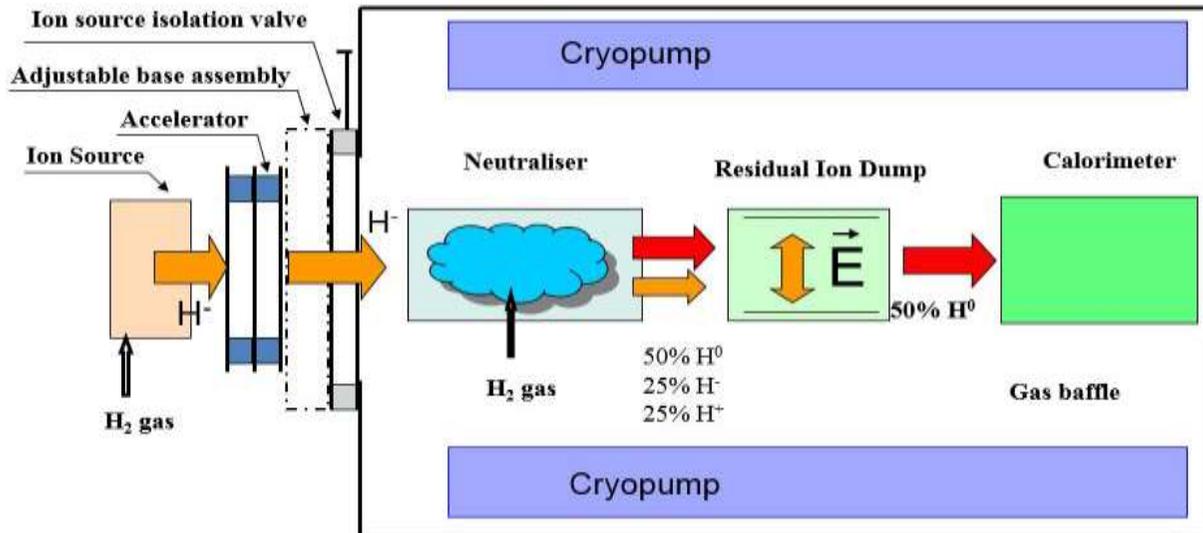


## Beam source (BS)

- ▶ Half size of ITER beam source with **1×4 RF drivers** and Cs seeding
- ▶ 1×200kV accelerator, not immersed in vacuum condition

## Beamline (BL)

- ▶ Full size of ITER beamline with a longer length, **gas neutralizer, electric deflection system**
- ▶ But **longer residual ion dump, longer and larger cryopump**
- ▶ Potential capability for full-size ITER beam source with 400keV accelerator



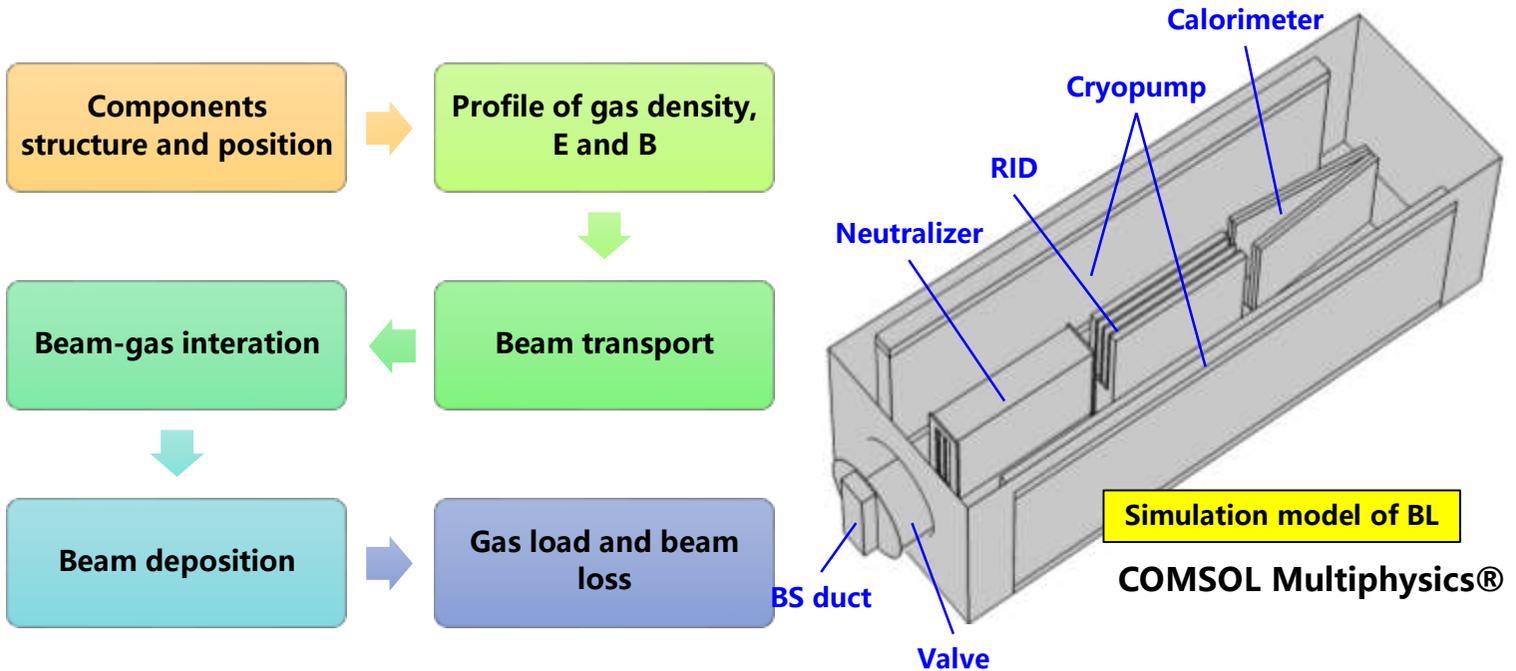


# Conceptual design of beamline



## Design process

- ▶ Optimize structure and position of inner components → **Minimize gas load on pump system and beam loss on inner components**



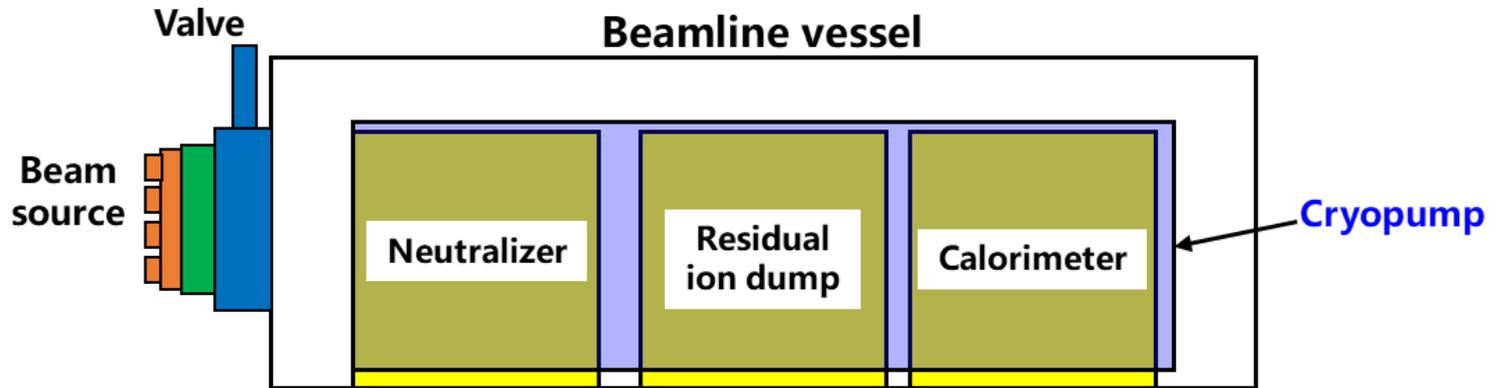


# Conceptual design of beamline



## Components structure and position (Version 1.0)

- ▶ **Beamline vessel (BLV):** rectangle, 12m long, 4m high, 4m wide
- ▶ **Neutralizer (NEU):** rectangle, 3m long, 1.7m high, 0.32m wide, 2 channels; Entrance is located 1m downstream of BLV front plate, exit is 0.5m upstream of RID
- ▶ **Residual ion dump (RID):** rectangle, 3m long, 1.7m high, 0.32 wide, 2 channels
- ▶ **Calorimeter (CAL):** V-shape, 5° opening, 3m long, 2m high, 0.2m downstream of RID exit
- ▶ **Cryopump:** two cryopumps; 10m long, 3m high, 0.45m deep; pumping surface 10m×2.5m





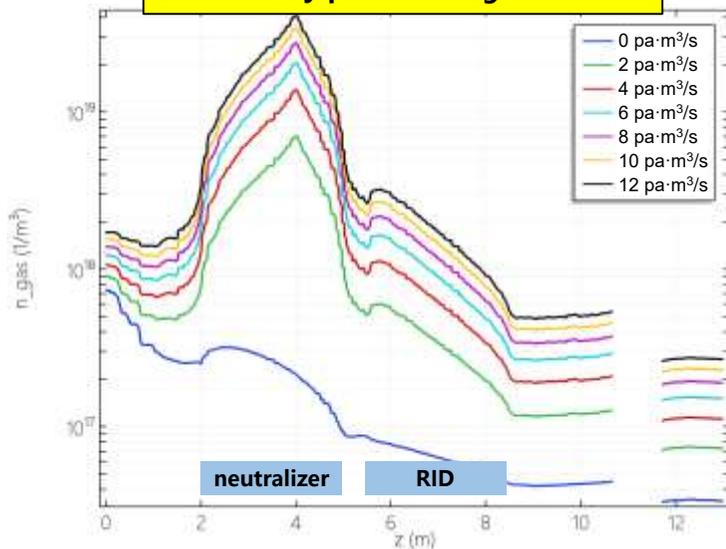
# Beam transport



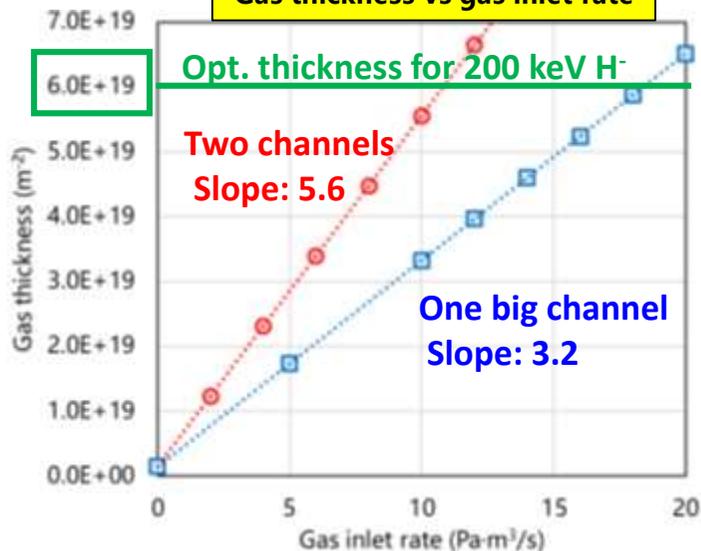
## Gas density profile

- ▶ Neutralizer is divided into **2 adjacent vertical channels** with central panel (**40mm wide**), to minimize gas inlet rate
- ▶ Residual gas ( $\sim 1000\text{K}$ ) from beam source is  $2\text{Pa}\cdot\text{m}^3/\text{s}$ , capture factor is 0.3 for each cryopumps surface

Gas density profile along centerline



Gas thickness vs gas inlet rate



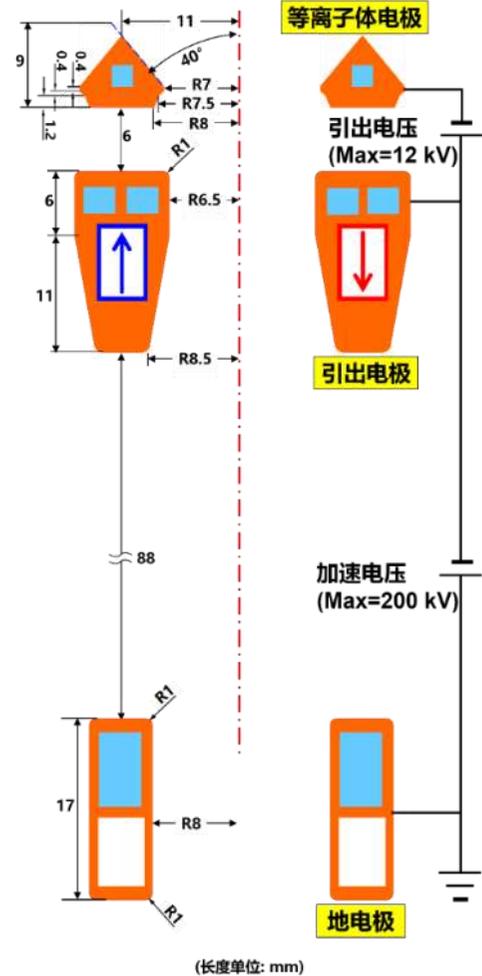
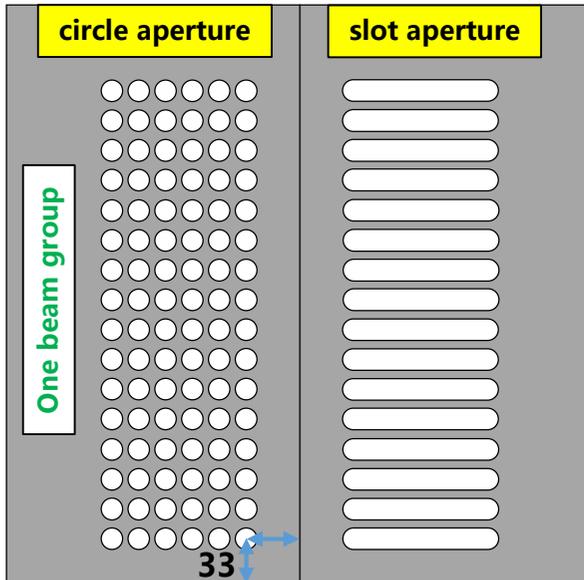


# Beam transport



## Ion beam emission from BS

- ▶ Apply the **ion beam simulation results** in accelerator, except a gaussian assumption
- ▶ Based on ITER-NBI accelerator: **2×4 beam groups, each 6×16 apertures, aperture spacing 22mm and 20mm**
- ▶ **Two GG designs:** (1) circle aperture, (2) slot aperture for less stripping loss





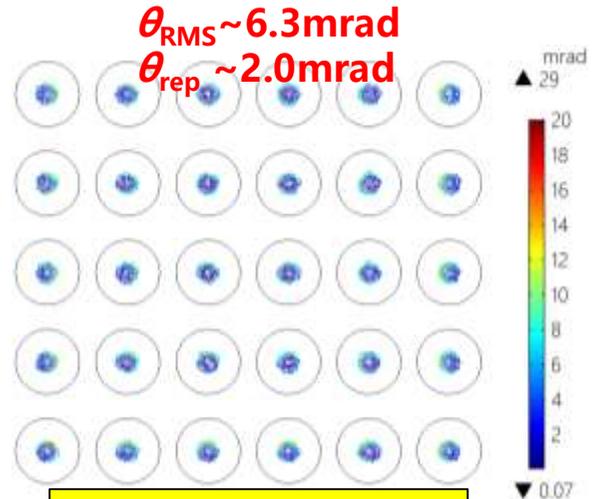
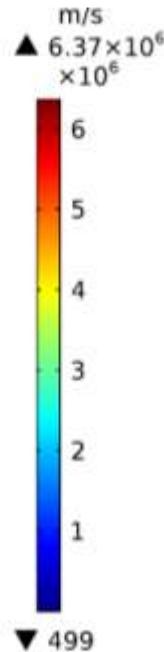
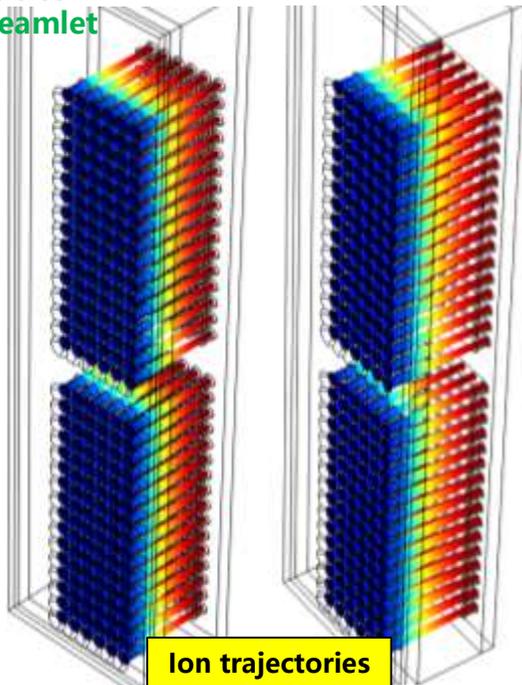
# Beam transport



## Ion beam emission from BS

- ▶ 1/4 grids model for calculating ion trajectories
- ▶  $E_{\text{ext}}=9\text{kV}/7\text{mm}$ ,  $E_{\text{acc}}=200\text{kV}/80\text{mm}$ ,  $j_{\text{ext}}=300\text{A}/\text{m}^2$
- ▶ **No magnetic field, no beam steering and focusing**

1000 particles  
for one beamlet



Scatter diagram on GG exit





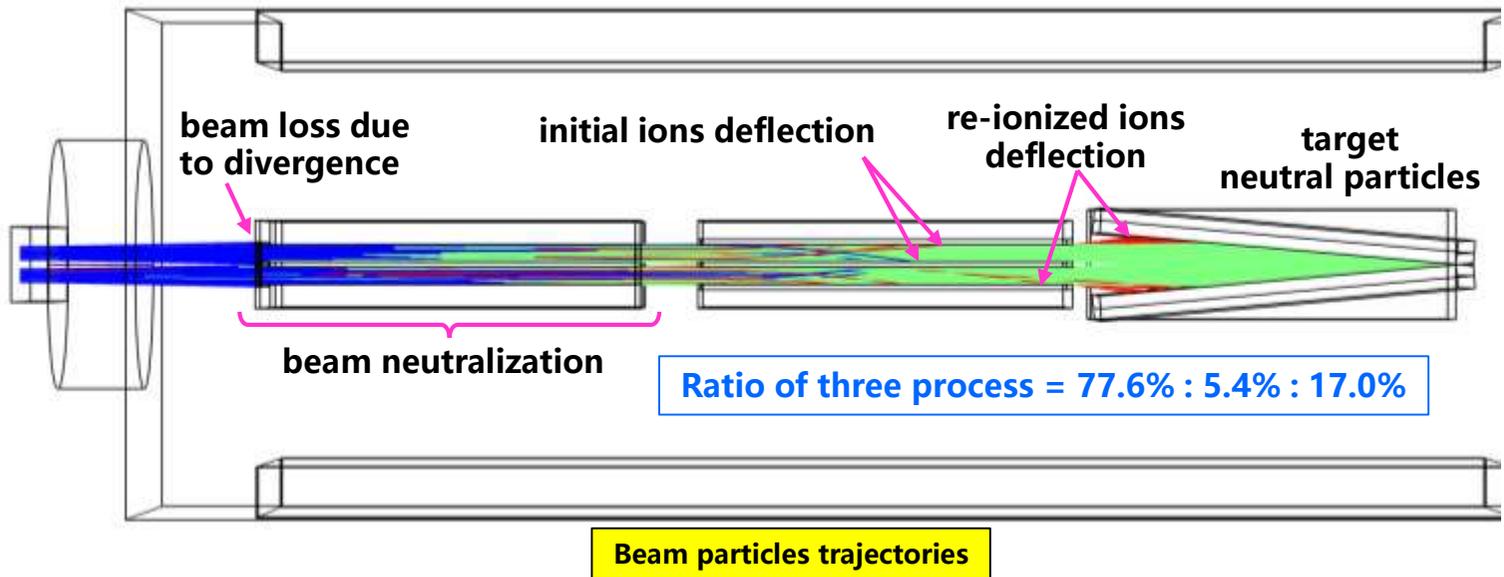
# Beam transport



## Beam transport along BL

- ▶ 12 middle beamlets **from circle GG apertures** are used to display beam transmission
- ▶ **Step length is ~5cm** for judgement of once beam-gas collision
- ▶ **Blue**—negative ions, **Green**—neutral particle, **Red**—positive ions

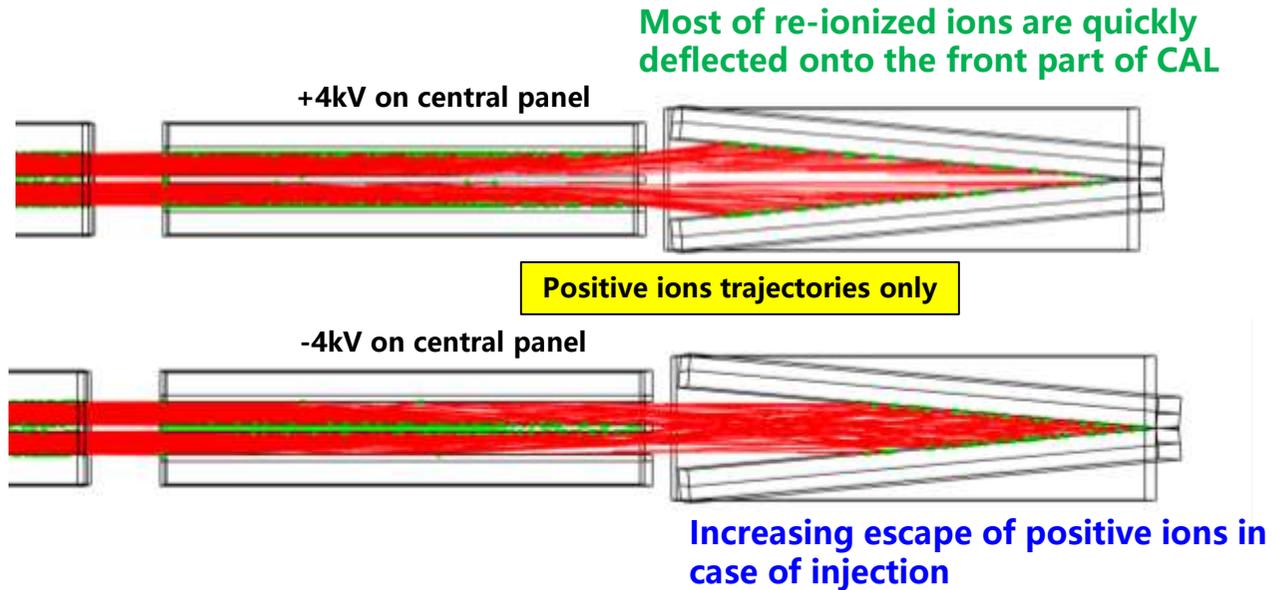
	beam-gas process	cross-section (200keV H)
1.	$H^- + H_2 \rightarrow H^0 + e + H_2$	2.51E-20
2.	$H^- + H_2 \rightarrow H^+ + 2e + H_2$	1.76E-21
3.	$H^0 + H_2 \rightarrow H^+ + e + H_2$	7.04E-21





## Beam separation

- ▶ Positive or negative voltage apply on central panel of RID (**40mm wide**)?  
(i.e., central panel collect negative ions or positive ions?)
- ▶ The largest difference is **the loss positions of positive ions on CAL**



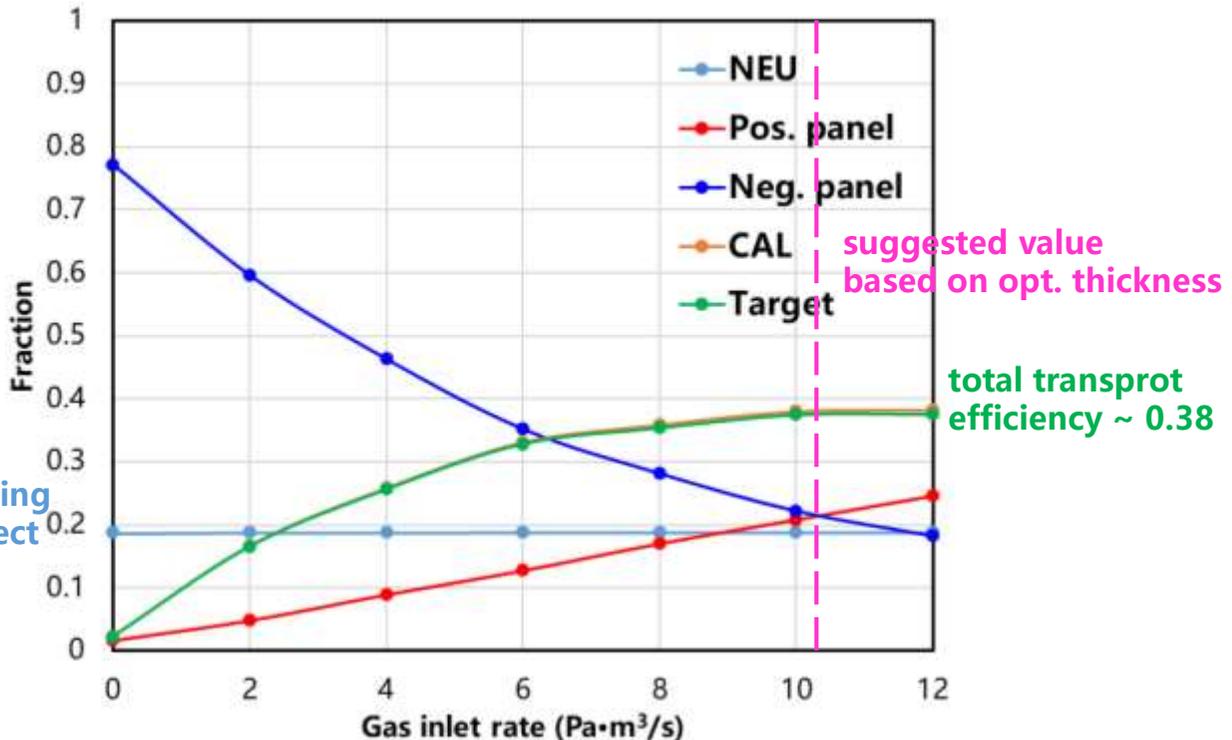


# Beam transport



## Distribution of beam deposition

- ▶ Due to transmission loss and re-ionization loss, **target neutral particles are less than expected**
- ▶ To attain the target neutral beam power of >2MW, **200keV@26A H- ion beam** is required

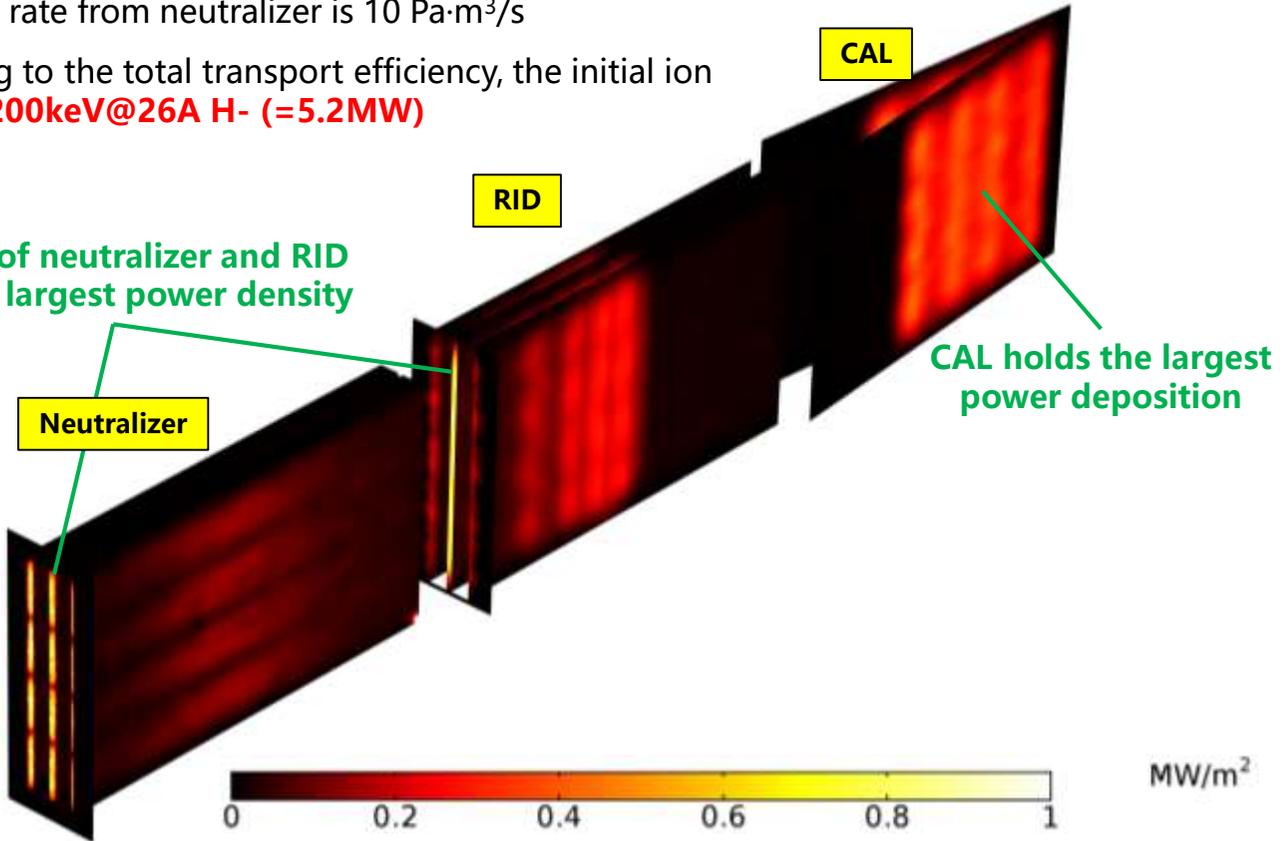




## Distribution of power deposition

- ▶ Gas inlet rate from neutralizer is  $10 \text{ Pa}\cdot\text{m}^3/\text{s}$
- ▶ According to the total transport efficiency, the initial ion beam is **200keV@26A H- (=5.2MW)**

Entrance of neutralizer and RID holds the largest power density



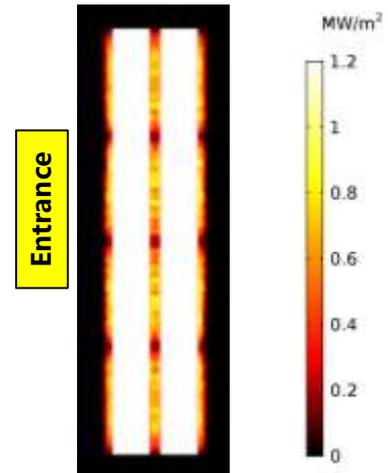


# Beam deposition

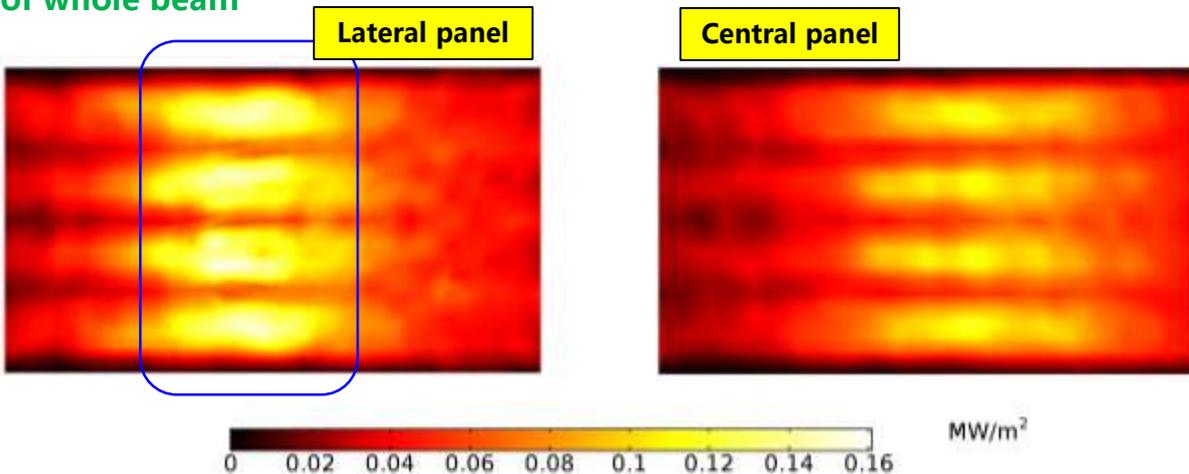


## Power deposition on neutralizer

- ▶ **Leading edge of panels** intercepts a high power density up to  $1.2\text{MW/m}^2$
- ▶ Without beam focusing, **beam loss due to repulsion** accumulate in the middle section of lateral panel
- ▶ Power density on central panel is lower, but is still unacceptable



## Beamlets from lateral aperture column of whole beam





# Beam deposition

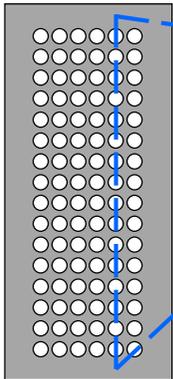


## Beam deposition on RID

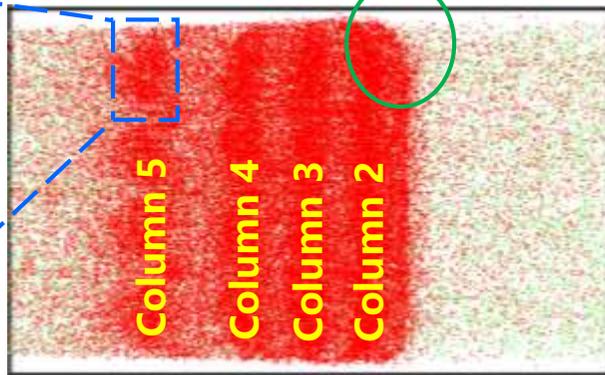
- ▶ **Difference of envelope** is caused by edge field effect
- ▶ Separations of footprints are relevant to beamlet groups and aperture column

**Footprints from aperture column No.1 and 6 are not clear due to interception loss**

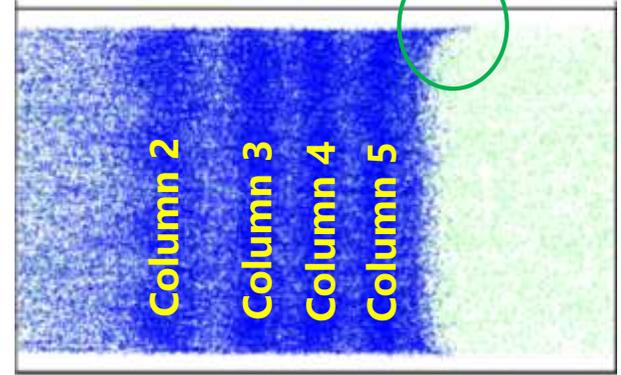
one beam group



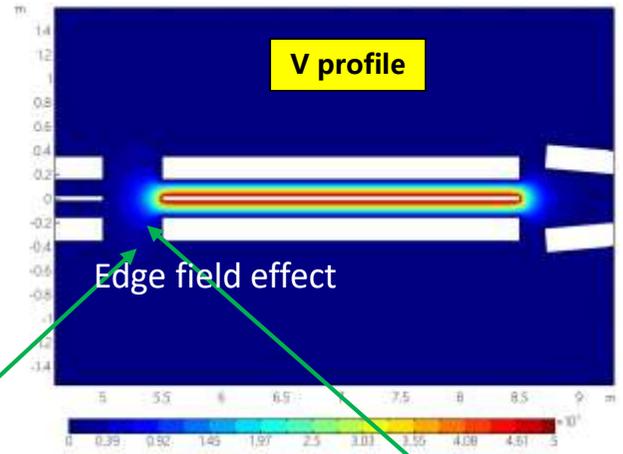
Aperture column No. 1~6



Lateral panel



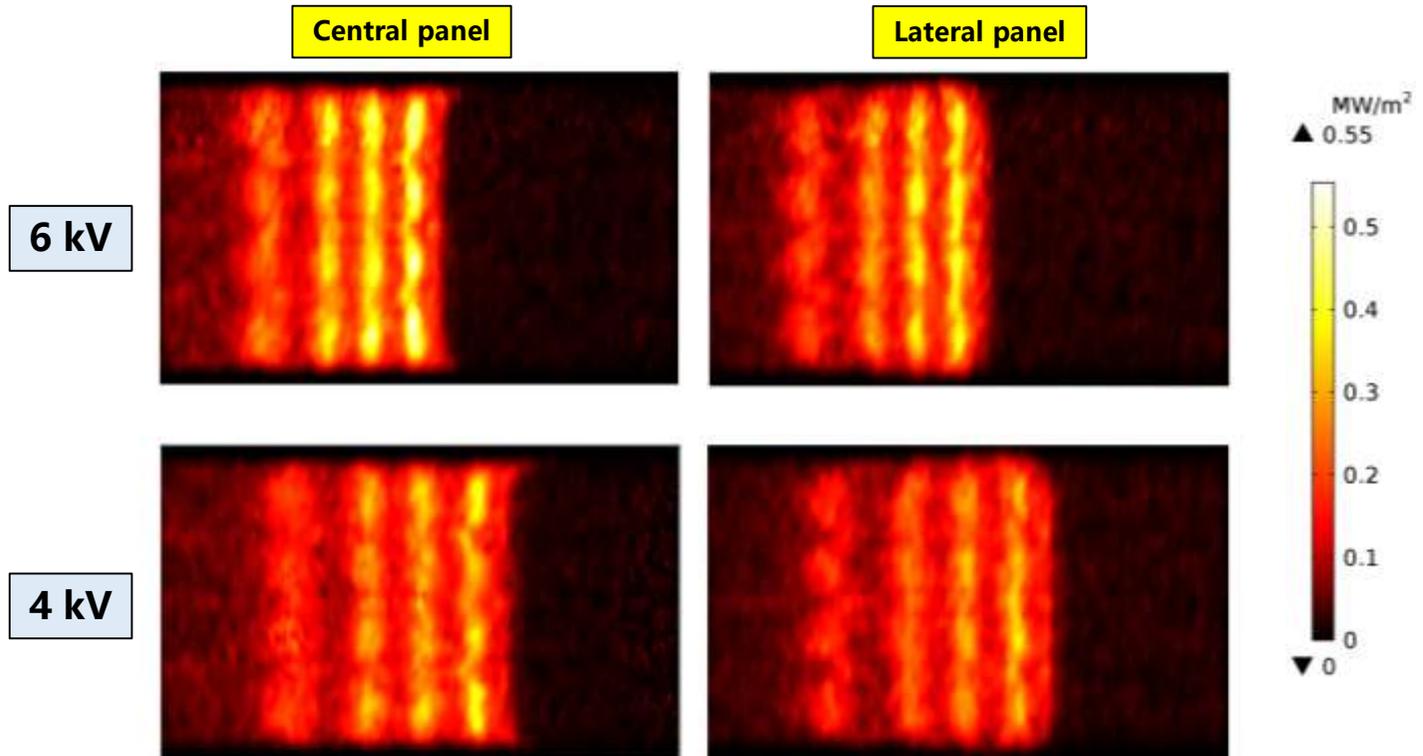
Central panel





## Power deposition on RID

- ▶ Sweeping the additional voltage is to decrease the maximum average power density
- ▶ Larger additional voltage  $\rightarrow$  closer to entrance, higher power density



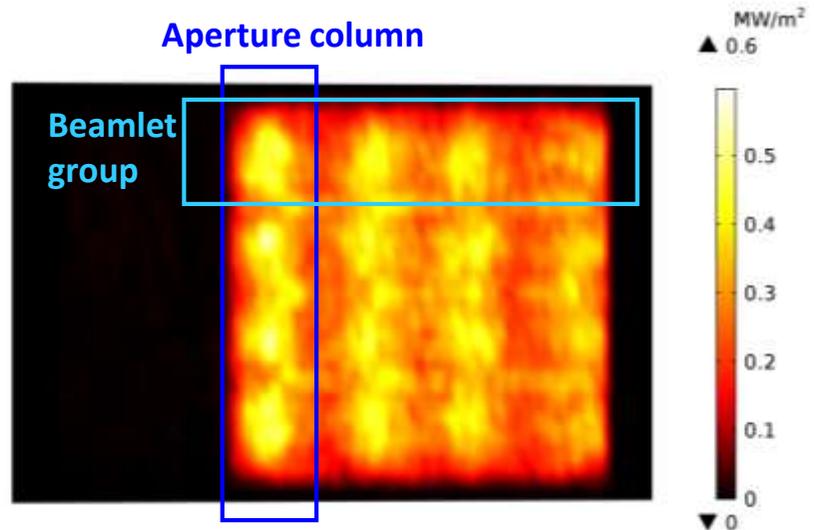
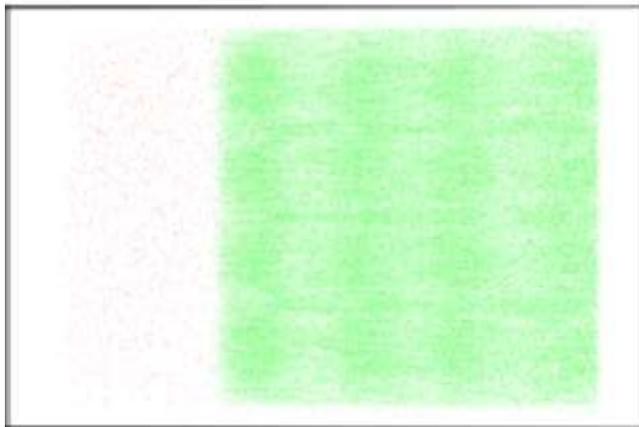


# Beam deposition



## Beam deposition on calorimeter

- ▶ Separations of beamlet groups in vertical direction and aperture column in horizontal direction are also clear on CAL
- ▶ Loss of beamlets from aperture column No. 1 and 6 suggests **the beam focusing is required**





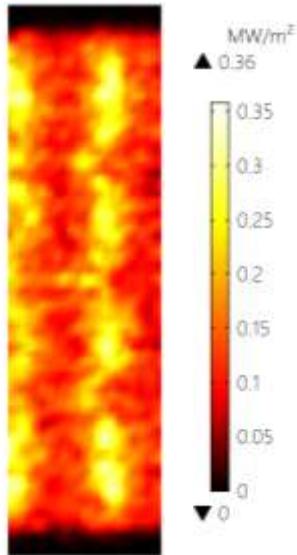
# Thermo-mechanical analysis



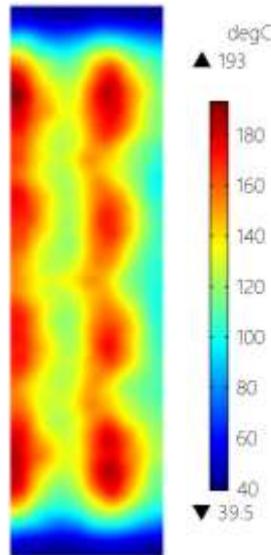
## Central panel of RID (based on results of power deposition)

- ▶ Each RID panel is made of several elements with the width of 0.5m, each element is embedded with 5 cooling channels

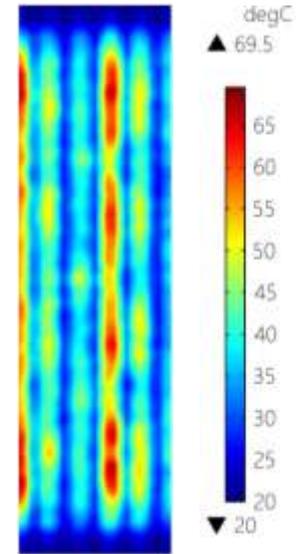
dia.=20mm  
inlet pressure=0.4 MPa



Power density



Without cooling



With cooling

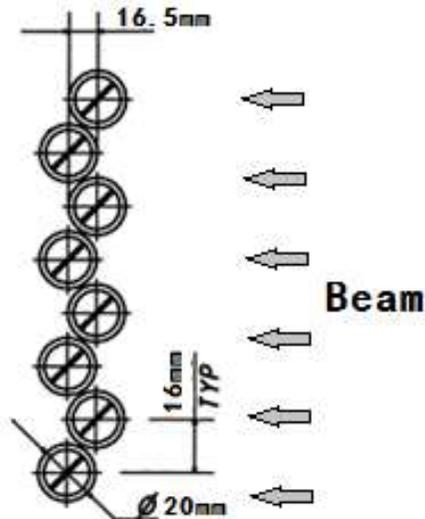


# Thermo-mechanical analysis

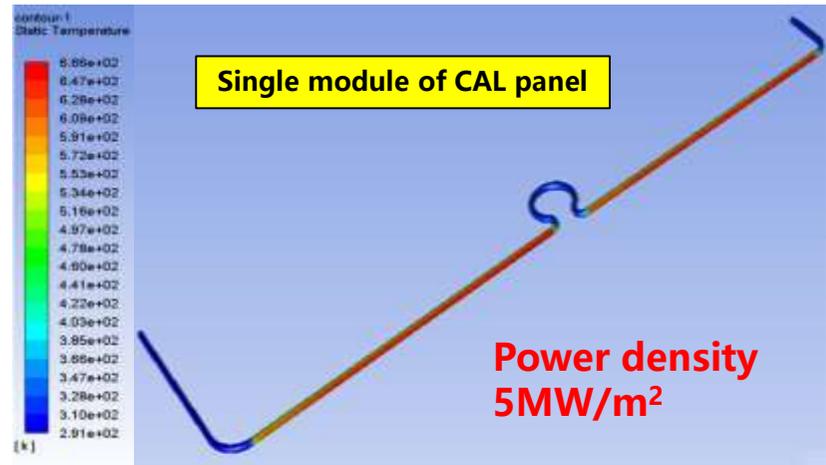


## Calorimeter panel (helpless to complex structure)

- ▶ made up of a double array of **swirl tubes**
- ▶ Ansys® software is used to apply the thermo-mechanical analysis



Conceptual design of swirl tubes





# Summary



- CFETR is a transition fusion reactor between ITER and DEMO, initial requirement of **NB heating for CFETR:  $>0.8\text{MeV}$ ,  $>30\text{MW}$ ,  $>1\text{h}$**
- National project of CFETR-NBTF has be started, target value:  **$\text{H}^0$  beam  $200\text{keV}$ ,  $>2\text{MW}$ ,  $3600\text{s}$**
- Conceptual design of beamline is ongoing, **a model of beamline has been developed to analyze beam transport and interaction with components**
- This model can give the distribution of gas density, E and B
- Beam transport is based on negative ions orbits from a real accelerator
- Beam neutralization and separation are simulated, and a total transmission efficiency of 0.38 is derived without beam steering and focusing
- Beam footprints on RID and CAL are separated by beam groups and aperture column



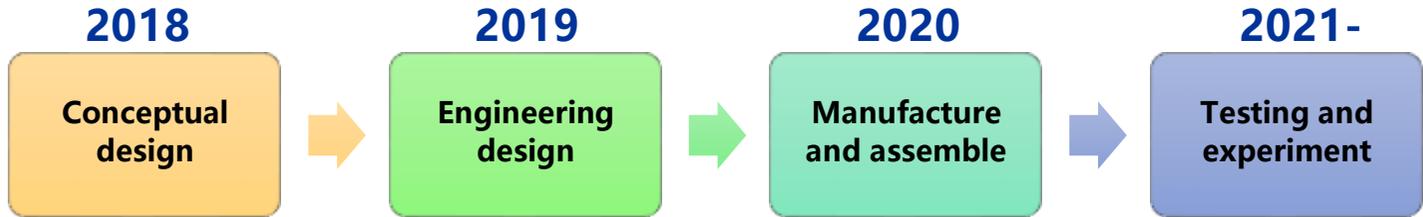
# Next step of model



- **Update the beam optics** according to an actual design of grids system, including magnetic field, beam steering and beam focusing
- Carry out the **design of electron dump** to intercept stray particles from beam source and to protect the cryopump
- Optimize the **design of beam collimators** along the beam transport, to minimize and uniform the deposition power density
- Apply the **thermo-structural analysis to key components** based on the results of beam deposition
- Design the **thermocouple array** (position and number) based on the results of beam transport, to measure the beam profile and divergence



## Development schedule

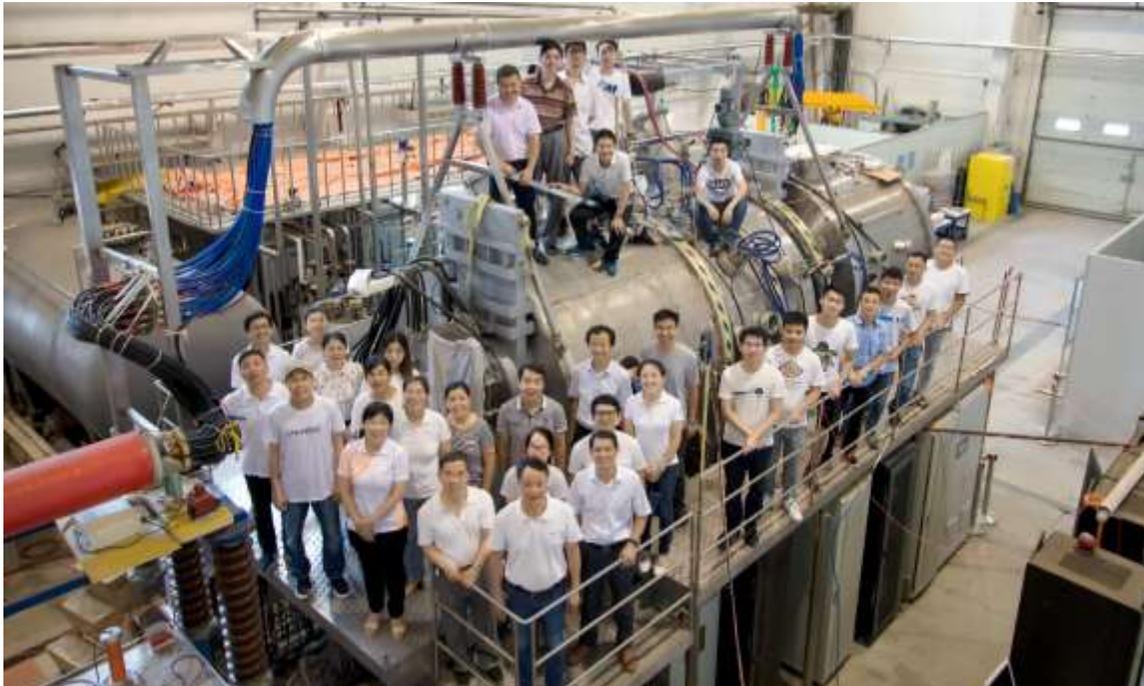


## Beyond beamline development

- ▶ Research on **plasma/laser neutralizer** for negative ions, based on the RF negative ion source test equipment at ASIPP
- ▶ Research on **enhanced heat transfer technology** for high-power and steady-state beamline components, based on high heat flux test bed at ASIPP



# Thanks for your attention



**ASIPP-NBI team**